

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 0 3 9 7										PAGE (3) 1 OF 3																																		
TITLE (4) RPS Actuations During Reactor Recirculation Pump Testing																																																						
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																								
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES										COCKET NUMBER(S)																	
0 6			0 6			8 5			8 5			0 4			0 0			0 0			7 0			3 8			5													0 5 0 0 0 0														
0 6			0 6			8 5			8 5			0 4			0 0			0 0			7 0			3 8			5													0 5 0 0 0 0														
OPERATING MODE (9) 4										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																																												
POWER LEVEL (10) 0 0 1 0										20.402(b)										20.406(e)										X 80.731(e)(2)(iv)										73.71(b)														
										20.406(a)(1)(i)										80.36(e)(1)										80.731(e)(2)(v)										73.71(c)														
										20.406(a)(1)(ii)										80.36(e)(2)										80.731(e)(2)(vi)										X OTHER (Specify in Abstract below and in Text, NRC Form 366A)														
										20.406(a)(1)(iii)										80.731(e)(2)(i)										80.731(e)(2)(vii)(A)										50.72(b)(2)(ii)														
										20.406(a)(1)(iv)										80.731(e)(2)(ii)										80.731(e)(2)(viii)(B)																								
										20.406(a)(1)(v)										80.731(e)(2)(iii)										80.731(e)(2)(ix)																								
LICENSEE CONTACT FOR THIS LER (12)																																																						
NAME R. L. Koenigs, Compliance Engineer																				TELEPHONE NUMBER 5 0 1 9 3 1 7 7 1 2 5 0 1 1																																		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2279																																																						
CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRC					CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRC									
SUPPLEMENTAL REPORT EXPECTED (14)																				EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR																								
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO																																		
ABSTRACT (Limit to 1400 spaces (i.e., approximately fifteen single-space typewritten lines) (16))																																																						
<p>On 6/6/85, during testing following repairs to Reactor Recirculation System (RRC) Pump 1B, two Reactor Protection System (RPS) actuations occurred. Each RPS actuation occurred while the Plant was shutdown and was the result of an RRC Pump 1B (RRC-P-1B) testing evolution.</p> <p>Reactor Pressure Vessel (RPV) level was being maintained at +30 in. and when RRC-P-1B was started an apparent inventory loss resulted and RPV level decreased to +13 in. This resulted in an RPS actuation.</p> <p>Later in the test sequence as RRC-P-1B was transferred from its 15 Hz power supply to a 60 Hz power supply, another RPS actuation was incurred. RRC-P-1B is supplied with a 8900 hp motor and the starting inrush current resulted in decreased RPS bus "A" voltage. This decreased bus voltage caused RPS bus "A" undervoltage relays to initiate a "half scram" condition. Since the opposite division was already in a "half scram" condition due to a main steam line high radiation surveillance test, a full RPS actuation occurred.</p> <p>The RRC-P-1B test procedure used was a temporary procedure and is no longer approved for use.</p>																																																						
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7 8 5	-	0 4 0	-	0 0 0 2	OF 3

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

- a) Power Level - 0%
- b) Reactor Mode - 4 (Cold Shutdown)

Event

On 6/6/85, during testing following repairs to Reactor Recirculation Pump 1B (RRC-P-1B), two Reactor Protection System (RPS) Actuations occurred. Each RPS actuation occurred while the Plant was shutdown and was the result of an RRC-P-1B evolution.

The first event occurred at 0115 hours with RPV level being maintained at +30 inches. When RRC-P-1B was started a sudden observed level decrease occurred. The RRC pump takes suction from the annulus area of the RPV and the sudden decrease in observed level was due to the amount of water required to fill the steam separators before water could return to the annulus region. This resulted in RPV level decreasing to +13 inches and initiating a RPS actuation.

At 0618 hours, during the same test procedure, RRC-P-1B was transferred from its 15 Hz power supply to a 60 Hz power supply. The RRC-P-1B motor is 8900 hp and the starting inrush current, combined with existing Plant electrical loading, resulted in decreased voltage on 480 VAC motor control center MCC-6B. MCC-6B and RRC-P-1B are both supplied from the same 6.9 kv switchgear (SH-6). Since the RPS motor generator set "A" was secured for modification, the alternate power supply MCC-6B, which was being fed from the startup transformer, was supplying power for the RPS "A" trip system. The decrease voltage on MCC-6B caused the RPS "A" undervoltage relays to deenergize. This resulted in a "half scram" condition. Since the opposite RPS trip system was already in a "half scram" condition due to main steam line high radiation surveillance testing, a full RPS actuation occurred.

Immediate Corrective Action

- o Following the first RPS actuation, RPV indicated level was restored to above the RPS trip point, the trip signal was reset and testing continued.
- o Following the second event, the RPS "A" trip signal was reset and testing continued.

Further Corrective Action

- o The test procedure governing the RRC-P-1B testing was a temporary use procedure and was cancelled subsequent to these events. No further action is anticipated as this testing is not expected to reoccur and startup transformer loading limitations are adequately addressed in permanent Plant procedures.

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TEXT (if more space is required, use additional NRC Form 288A's) (17)

Safety Significance

No actual RPV inventory loss occurred during the first event and the RPS bus "A" voltage decrease during the second event was only momentarily and would not have occurred in the normal Plant lineup. In both cases the RPS responded per its design. There was no threat to the safety of the Plant, its personnel or the public as a result of these events.

Similar Events

See LER 84-079-01 for a discussion on startup transformer loading limitations.

EIIS Information

Text Reference

EIIS Reference

RRC-P-1B

AD

P

RPS

JC

MCC-6B

EC

BU

SH-6

EA

BU

System Component

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

July 3, 1985

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

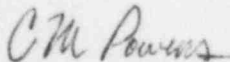
Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-040

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-040 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 0135 hours and 0756 hours on 6/6/85.

Very truly yours,



C.M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:1a

Enclosure:

Licensee Event Report No. 85-040

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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