NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	1 OF 0 12

TITLE (4)

ESF Actuation - Control Room Ventilation Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 6	0 2	8 5	8 5	0 3 7	0 0	0 7	0 1	8 5		0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
3	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services	AREA CODE 3 1 6 3 6 4 - 8 8 3 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	J E	R E	G 0 6 3	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On three different occasions, an Engineered Safety Features Actuation Signal was initiated by a control room intake radiation monitor spurious alarm causing a Control Room Ventilation Isolation Signal. The incidents occurred at 1304 CDT on June 2, 1985, 0023 CDT on June 5, 1985, and 0257 CDT on June 7, 1985. All required engineered safety features equipment responded properly on each occasion.

Prior to each of these incidents, the plant was in Mode 3, Hot Standby, with the Reactor Coolant System at normal operating pressure and temperature.

On each occasion, no radiation above normal background was present, as determined by a redundant radiation monitor. No damage to plant equipment occurred as a result of these incidents and at no time did conditions develop which could have threatened the public health or safety.

Previous actuations due to spurious alarms from this monitor were discussed in LER 85-013-00.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	8 5	— 0 3 7	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On three different occasions, an Engineered Safety Features Actuation Signal was initiated by a spurious electronic "spike" in a control room intake radiation monitor (GK-RE-04). The incidents occurred at 1304 CDT on June 2, 1985, 0023 CDT on June 5, 1985, and 0257 CDT on June 7, 1985. Each of these spikes resulted in a Control Room Ventilation Isolation Signal (CRVIS), in which all required engineered safety features equipment responded properly.

Prior to each of these incidents, the plant was in Mode 3, Hot Standby, with the Reactor Coolant System at normal operating pressure and temperature.

In each instance, no radiation above normal background was present, as determined by redundant radiation monitor GK-RE-05, and the actuated systems were restored to a normal configuration per plant operating procedures.

Subsequent investigation of each incident identified a mismatch between the software and hardware in the RM-80 microprocessing unit for the radiation monitor as the probable cause of the spurious alarms. Prior CRVIS actuations attributable to this mismatch were identified via LER 85-013-00. As stated in LER 85-013-00, resolution of the mismatch for the radiation monitors is expected from the manufacturer in the near future and will be installed when it becomes available.

No damage to plant equipment occurred as a result of these incidents and at no time did conditions develop which could have threatened the public health or safety.

KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

July 1, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-170
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-037-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

C C Mason

for Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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