

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
La Crosse Boiling Water Reactor (LACBWR)DOCKET NUMBER (2)
0 5 0 0 0 4 0 9PAGE (3)
1 OF 0 2TITLE (4)
Reactor Scram due to Low Water Level

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | | | | | | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---|------------------|---|---|---|---|---|---|---|------|-----------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | DOCKET NUMBER(S) | | | | | | | | | |
| 0 | 4 | 2 | 7 | 8 | 5 | 8 | 5 | 0 | 1 | 1 | 0 | 0 | 0 | 5 | 2 | 0 | 8 | 5 | None | 0 5 0 0 0 |

| OPERATING MODE (9) | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11) | | | | | | | | | |
|--------------------|-------|---|--|------------------|---|----------------------|--|--|--|--|--|
| 1 | | 20.402(b) | | 20.405(c) | X | 50.73(a)(2)(iv) | | 73.71(b) | | | |
| | | 20.405(a)(1)(i) | | 50.38(c)(1) | | 50.73(a)(2)(v) | | 73.71(c) | | | |
| POWER LEVEL (10) | 0 7 8 | 20.405(a)(1)(ii) | | 50.38(c)(2) | | 50.73(a)(2)(vii) | | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | | | |
| | | 20.405(a)(1)(iii) | | 50.73(a)(2)(i) | | 50.73(a)(2)(viii)(A) | | | | | |
| | | 20.405(a)(1)(iv) | | 50.73(a)(2)(ii) | | 50.73(a)(2)(viii)(B) | | | | | |
| | | 20.405(a)(1)(v) | | 50.73(a)(2)(iii) | | 50.73(a)(2)(ix) | | | | | |

LICENSEE CONTACT FOR THIS LER (12)
NAME
Lynne S. Goodman, LACBWR Operations EngineerTELEPHONE NUMBER
AREA CODE
6 0 8 6 8 9 - 2 3 3 1

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | |
|--|-----------|-----------|--------------|---------------------|-------|--------|-----------|--------------|---------------------|
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPDOS | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPDOS |
| X | J B T C | | F 1 8 0 N | | | | | | |
| X | J B A M P | | F 1 8 0 N | | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

Reactor scrambled due to low water level when oncoming reactor feedwater pump (RFP) did not respond properly during swap of operating feedwater pumps necessitated by the erratic control exhibited by operating pump. Both High Pressure Core Spray Pumps and Emergency Diesel Generators started. The Shutdown Condenser initiated. Emergency equipment was secured.

The dead band width and gain were adjusted on the 1B RFP (operating pump) controller. The 1A RFP (oncoming pump) controller amplifier was replaced and the Reactor Feedwater Flow Controller response was adjusted.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

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|--|--|----------------|-------------------|-----------------|----------|----|-----|
| FACILITY NAME (1) La Crosse Boiling Water Reactor | DOCKET NUMBER (2) 0 5 0 0 0 4 0 9 8 5 - 0 1 1 - 0 0 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | | | | 0 2 | OF | 0 2 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During midshift on April 27, 1985, water level control (JB) exhibited erratic behavior, with feedwater (SJ) flow varying from 410,000 lb/hr to 490,000 lb/hr. The reactor (RCT) was operating at approximately 78% Rated Thermal Power. It was decided to switch to the other Reactor Feedwater Pump (RFP)(P).

During normal operation, 1 of 2 RFP's is running. Pump flow is controlled by varying the amount of coupling between a constant speed motor (MO) and the pump. The reactor water level control system is a 3 element system which compares reactor water level, steam flow and feedwater flow. The Reactor Feedwater Flow Controller send signals to the operating RFP Hydraulic Coupling (CPLG) Controller to increase or decrease the amount of coupling and hence pump speed and feedwater flow. Normally, the feedwater flow varies slightly during operation.

At 0714, the 1A RFP hydraulic coupling pump and the 1A RFP were started. The 1A and 1B RFPs were paralalled in automatic control. The operators put 1B RFP to manual and started backing it down, but the 1A RFP did not pick up the load as it should have. The opertors started increasing the speed on the 1B RFP, but the reactor scrammed on low water level, before the 1B RFP responded. The nominal low water level scram setpoint (JC) is 12 inches below normal operating level.

All emergency systems functioned properly. The Main Steam (SB) Isolation Valve (ISV) closed, as did other Containment Building isolation valves (JM). Both High Pressure Core Spray (HPCS)(BG) Pumps and both Emergency Diesel Generator (EDG)(EK)(DG) started. The Shutdown Condenser (SDC)(BL) initiated. Water level increased rapidly to about 20 inches above normal due to the cessation of steam flow, initiation of the HPCS pumps, and swell due to the SDC coming into service. The HPCS pumps, SDC, and EDGs were removed from service. Manual blowdown to the main condenser (SG) was established to control water level at 0730.

The Reactor Feedwater Flow Controller reset time was reduced. The dead band width and gain were adjusted on the 1B RFP Coupling Controller amplifier (AMP). The 1A RFP amplifier was checked out and appeared to operate satisfactorily, but was replaced with a spare as a precaution. During the subsequent startup, the 1A RFP was used. Water level control was tighter than before the scram and appeared near optimum. Optimum system control is achieved when feedwater flow varies only slightly without requiring constant adjustment of the amount of coupling.



LA CROSSE BOILING WATER REACTOR (LACBWR) • P.O. BOX 275
GENOA, WISCONSIN 54632 • (608) 689-2331

May 20, 1985

In reply, please
refer to LAC-10766

DOCKET NO. 50-409

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: DAIRYLAND POWER COOPERATIVE
LA CROSSE BOILING WATER REACTOR (LACBWR)
PROVISIONAL OPERATING LICENSE NO. DPR-45
LICENSEE EVENT REPORT NO. 85-11

Reference: 10 CFR 50.73

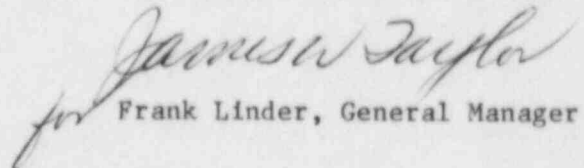
Gentlemen:

In accordance with 10 CFR 50.73, attached is Licensee Event Report
No. 85-11.

If there are any questions, please contact us.

Sincerely,

DAIRYLAND POWER COOPERATIVE

for 
Frank Linder, General Manager

FL:LSG:sks

Attachment

cc: J. G. Keppler, NRC Region III
NRC Resident Inspector
D. Sherman, ANI Library
Richard Dudley, LACBWR Project Manager
INPO

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WP6.20.1