

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Waterford 3 Steam Electric StationDOCKET NUMBER (2)
0 5 0 0 0 3 8 1 2 1 OF 0 3TITLE (4)
Pressurizer Heater Capacity Less Than Technical Specification LimitEVENT DATE (5)
MONTH DAY YEAR
0 4 2 5 8 5
LER NUMBER (6)
YEAR SEQUENTIAL NUMBER REVISION NUMBER
8 5 0 1 6 0 0
REPORT DATE (7)
MONTH DAY YEAR
0 5 2 4 8 5
OTHER FACILITIES INVOLVED (8)
FACILITY NAMES
N/A
DOCKET NUMBER(S)
0 5 0 0 0 0 0 0 0 0OPERATING MODE (9)
1
POWER LEVEL (10)
0 1 5 1 0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)
20.402(b)
20.405(a)(1)(i)
20.405(a)(1)(ii)
20.405(a)(1)(iii)
20.405(a)(1)(iv)
20.405(a)(1)(v)
20.406(c)
50.36(c)(1)
50.36(c)(2)
50.73(a)(2)(i)
50.73(a)(2)(ii)
50.73(a)(2)(iii)
50.73(a)(2)(iv)
50.73(a)(2)(v)
50.73(a)(2)(vi)
50.73(a)(2)(vii)
50.73(a)(2)(viii)(A)
50.73(a)(2)(viii)(B)
50.73(a)(2)(ix)
73.71(b)
73.71(c)
OTHER (Specify in Abstract below and in Text, NRC Form 366A)LICENSEE CONTACT FOR THIS LER (12)
NAME
O.D. Hayes, Operations Superintendent
TELEPHONE NUMBER
AREA CODE
5 0 4 4 6 4 - 3 1 1 8COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NPDs
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NPDsSUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE)
X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

On April 25, 1985 Waterford 3 Steam Electric Station was at 50% reactor power when Operations Personnel discovered that one of the two banks of pressurizer heaters powered from the 1E bus, did not have a nominal capacity of 150 kW as required by Technical Specification 3.4.3b. An immediate investigation revealed that Operations Personnel removed from service what was thought to be a spare breaker located in Power Distribution Panel 376B. However, due to a drawing discrepancy, this breaker was in actuality the loaded breaker. Since one of the heaters was not operable, the capacity of the second bank of proportional heaters was about 50 kW less than that required by Technical Specifications.

The drawing discrepancy has since been corrected.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
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TEXT (If more space is required, use additional NRC Form 365A's) (17)

NARRATIVE

On April 25, 1985 Waterford 3 Steam Electric Station was at 50% reactor power when Operations Personnel were performing procedure OP-903-097, Pressurizer Heater Capacity Verification. The procedure is used to ensure heater operability in accordance with Technical Specifications 4.4.3.2. As described in Technical Specification 3.4.3b, at least two groups of pressurizer heaters powered from a Class 1E bus must have a nominal capacity of 150 kW. As a result of the test, Operations Personnel discovered that, although the capacity of bank 1 was 150 kW, the capacity of bank 2 was about 100 kW. An immediate investigation revealed that on March 31, 1985, because of an arcing problem, the spare breaker, as shown on Power Distribution Motor Data drawing B-289 sheet 102, located in Power Distribution Panel 376B, was removed from service for maintenance. However, after completion of OP-903-097, Plant Personnel discovered that the breaker was listed as the loaded breaker on Control Wiring Diagram B-424 sheet 292. A field verification revealed that the latter drawing was correct, and that the breaker was tagged out per the B-289 drawing. Since the breaker removed from service was in actuality the loaded breaker, the second bank of proportional heaters did not have the capacity required by Technical Specification.

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in the loss of 50 kW of heater capacity in proportional bank 2 from March 21 to April 25, 1985. Although the Technical Specifications require a total heater capacity of 300 kW for the two banks powered from a Class 1E bus, the safety of the plant was never in jeopardy since Waterford 3 had more than enough heater capacity to satisfy the natural circulation and loss of off-site power design analysis described in section 5.4.10.2 of the Final Safety Analysis Report.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 386A's) (17)

CORRECTIVE ACTION

Condition Identification Work Authorization 017422 and 017688, along with Station Modification 904, have been issued to correct the above problem. In addition, the Control Wiring Diagrams related to the pressurizer heaters have been field verified to ensure accuracy.

SIMILAR EVENTS

NONE

PLANT CONTACT

O.D. Hayes, Operations Superintendent, 504/464-3118



MIDDLE SOUTH
UTILITIES SYSTEM

LOUISIANA
POWER & LIGHT

142 DELARONDE STREET • P.O. BOX 8008
NEW ORLEANS, LOUISIANA 70174-8008 • (504) 388-2345

May 24, 1985

W3P85-1296
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-016-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(i).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), E.L. Blake,
W.M. Stevenson

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