

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Duane Arnold Energy Center										DOCKET NUMBER (2) 0 5 0 0 0 3 3 1 1										PAGE (3) 1 OF 0 2	
TITLE (4) Inadvertent Group III Isolation																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES None			DOCKET NUMBER(S) 0 5 0 0 0									
0 4	2 5	8 5	8 5	0 1 2	0 0	0 5	2 4	8 5				0 5 0 0 0									
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																			
POWER LEVEL (10) 0 0 0		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)									
		20.406(a)(1)(i)				50.36(c)(1)				<input type="checkbox"/> 50.73(a)(2)(v)		73.71(c)									
		20.406(a)(1)(ii)				50.36(c)(2)				<input type="checkbox"/> 50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 306A)									
		20.406(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)											
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)											
		20.406(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME James R. Probst, Technical Support Engineer								TELEPHONE NUMBER AREA CODE 3 1 9 8 5 1 - 7 4 5 8													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS												
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO											
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																					

On 4/25/85 at 0045 hours, with the reactor shutdown and defueled, a Group III Isolation and both Standby Gas Treatment Systems were auto-initiated on downscale Reactor Building vent radiation monitor trips. An Operator, after consulting an electrical distribution list, deenergized two Instrument AC circuits which provide power for several drywell cooling motor-operated valves upon which maintenance was to be performed. The list did not indicate to the Operator that the Reactor Building vent radiation monitors were also powered off these two circuits. The breakers were closed, and the isolations and initiations were reset by the Operators. The list consulted has been annotated to indicate the power source of the Reactor Building vent radiation monitors. As a long-term corrective action, a comprehensive Electrical Distribution Information System continues to be developed.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Duane Arnold Energy Center	DOCKET NUMBER (2) 0 15 0 0 0 3 3 1 8 5 - 0 1 2 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 4/25/85 at 0045 hours, with the reactor shutdown and defueled, a Group III Isolation (JM) and both Standby Gas Treatment Systems (BH), were auto-initiated on downscale radiation monitor (IL) trips. The trips were a result of maintenance work being performed on motor-operated valves used for drywell cooling (VB) whose control logic is powered off Instrument AC (ED).

In order to work on the motor operators, the circuit breakers for their power supplies were opened after an Operator had consulted an electrical distribution list in the Control Room. This list indicated to him that disconnection of these circuits would not have any undesirable consequences. Upon opening the circuits a Group III Isolation and Standby Gas Treatment System auto-initiation occurred as a result of Reactor Building vent shaft high radiation monitors failing downscale. These monitors are powered from the same circuits. The breakers were closed and the isolations and initiations were reset by the Operators. The valve motor-operator maintenance was rescheduled.

Investigation revealed that the electrical distribution list consulted by the Operator did not specifically state the Reactor Building vent high radiation monitors were powered off these circuits, although the list did provide as a reference an Operating Instruction which does state the power source of the monitors. As a short-term corrective action, the electrical distribution list consulted by the Operator has been annotated to include the information that the circuits are the power sources for the Reactor Building vent shaft monitors. As a long-term corrective action, an Electrical Distribution Information System is presently being developed by Iowa Electric. This system, which will eventually replace or augment lists consulted by Operators, is designed to provide a comprehensive load description for all plant circuits, including cautions regarding the consequences of circuit deenergization.

Iowa Electric Light and Power Company

May 24, 1985

DAEC-85- 0427

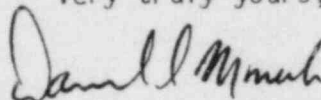
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Subject: Duane Arnold Energy Center
Docket No. 50-331
Op. License DPR-49
Licensee Event Report No. 85-012

Gentlemen:

In accordance with 10 CFR 50.73 please find attached a copy of the
subject Licensee Event Report.

Very truly yours,



Daniel L. Mineck
Plant Superintendent - Nuclear
Duane Arnold Energy Center

DLM/JRP/kp

attachment

cc: Mr. James G. Keppler
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

NRC Resident Inspector - DAEC

File A-118a

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May 24, 1985

TO:	L. Liu	K. Howard
	S. Tuthill	Operations Shift Supervisors
	R. McGaughy	DAEC Supervision - Routing Slip
	E. Root (Safety Committee)	STA Coordinator
	E. Matthews	INPO
	P. Ward	R. Salmon
	H. Rehrauer	T. Dalton
	D. Wilson	R. Lessly
	R. Hannen	P. Seckman
	K. Young	DAEC Commitment Control
	G. VanMiddlesworth	

FROM: D. Mineck
Plant Superintendent - Nuclear

FILE: A-118a

Please find attached one copy of a Licensee Event Report
that has been transmitted to the NRC.

Unique Report No.

Reportable Occurrence Report No. 85-012

Notification Letter No. DAEC-85-0427

DR Number 85-189

kp*