



GPU Nuclear, Inc.
Route 441 South
Post Office Box 480
Middletown, PA 17057-0480
Tel 717-944-7621

September 13, 1996

6710-96-2294

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for August 1996

Enclosed are two copies of the August 1996 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

A handwritten signature in dark ink, appearing to read "J. Knubel", is written over a faint circular stamp.

J. Knubel
Vice President and Director, TMI

WGH

cc: Administrator, Region I
TMI Senior Resident Inspector
96001

9609240187 960831
PDR ADOCK 05000289
R PDR

11
I 024

OPERATIONS SUMMARY

August 1996

The plant entered the month operating at 100% power and remained at that power level for the remainder of the month. Net unit electrical output averaged approximately 796 MWe during August.

MAJOR SAFETY RELATED MAINTENANCE

The major safety related maintenance items completed during the month involved the following equipment:

Pressurizer Pilot Operated Relief Valve RC-RV-2

The fuse clips for control circuit fuses FD-73 and FD-74 associated with the Pressurizer Pilot Operated Relief Valve (PORV) RC-RV-2 were replaced because the installed fuse clips were found bent.

Emergency Light Repairs

Repairs to Relay Room Emergency Light EL-L-19 and Reactor Building Personnel Access Hatch Emergency Light EL-L-102 were completed

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE September 13, 1996
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1
2. REPORTING PERIOD: AUGUST 1996
3. LICENSED THERMAL POWER: 2568
4. NAMEPLATE RATING (GROSS MWe): 872
5. DESIGN ELECTRICAL RATING (NET MWe): 819
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

THIS MONTH YR-TO-DATE CUMMULATIVE

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	5855.0	192840.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	5855.0	115398.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	5855.0	114236.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1,908,743.0	14,975,959.7	281,090,885.7
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	626,734.0	4,990,304.0	94,443,778.1
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	592,476.0	4,715,870.0	88,759,274.1
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	59.2
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	59.2
21. UNIT CAPACITY FACTOR (USING MDC NET)		101.3	102.5	58.6
22. UNIT CAPACITY FACTOR (USING DER NET)		97.2	98.3	56.2
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	34.7
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH): _____

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE ~~September 13, 1996~~
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: AUGUST

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
-----	--

1	797
2	798
3	798
4	795
5	795
6	793
7	794
8	793
9	796
10	800
11	803
12	803
13	803
14	798
15	795
16	798

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
-----	--

17	798
18	796
19	796
20	794
21	793
22	790
23	790
24	793
25	794
26	797
27	796
28	796
29	797
30	799
31	799

REPORT MONTH August 1996

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE September 13, 1996
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code ⁴ & *	Component Code ⁵ & *	Cause & Corrective Action to Prevent Recurrence
						None			

¹
 F Forced
 S Scheduled

²
 Reason
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

⁴
 Exhibit G - Instructions for
 preparation of Data Entry Sheets
 for Licensee Event Report (LER)
 File (NUREG-0161)

⁵ Exhibit I same source

⁶ Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 5, 1997**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **Yes. To support GPU Nuclear plans to do independent reload analyses for Cycle 12 as discussed in response to question 6 below, T.S. 6.9.5.2 would require revision to include references to the GPU Nuclear analysis methods applied to the reload.**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **A Technical Specification Change Request for the changes as discussed above will be submitted once the GPU Nuclear topicals are approved.**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

a) GPU Nuclear Letter 6710-96-2092, dated March 28, 1996 confirmed plans to perform independent reload design evaluations for Cycle 12, the next operation cycle, based on NRC approved methods described in GPU Nuclear Topical Reports TR-091A (core physics), TR-087 (core thermal hydraulics), TR-078 (FSAR safety analyses) and TR-092P (design and setpoints methodology) submitted to the NRC. The latter three are in the NRC review and approval stage.

At this time, completion of the NRC review and issuance of NRC SERs is active. We have received questions on all reports and are in the process of providing answers. All remaining reports are expected to be approved in an acceptable time frame to support our reload design activities.

The GPU Nuclear Cycle 12 reload program and results are expected to be available for NRC review in the March to April 1997 time frame.

b) Cycle 12 fuel rod performance calculations (e.g. internal pin pressure) will be performed by Framatome Cogema Fuels Company (FCF) using the approved TACO3 (BAW-10162P-A) and GDTACO (BAW-10184P-A) fuel codes. Results require minor changes to the Mark B9 fuel rod design (lower fill gas prepressure, increased plenum volume). The new design will meet all criteria in the latest approved revision of BAW-10179P-A, Safety Criteria and Methodology for Acceptable Cycle Reload Analyses. Fuel rod cladding corrosion calculations for all Cycle 12 fuel are being done by FCF using the COROS2 methodology now under review by the NRC with approval expected about September 1996. The TACO calculations are being done using power histories generated with the GPU Nuclear approved core physics codes CASMO-3/SIMULATE-3 (TR-091A). A letter requesting approval for use of the SIMULATE-3 power peaking uncertainty of 5.5% with the TACO methodologies, rather than the current FCF NEMO physics code (BAW-10180A, Rev 1) uncertainty of 4.8%, was submitted by FCF in August 1996 with approval requested by September 30, 1996 to support the Cycle 12 reload design schedule requirements.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 864
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through and beyond the end of the current operating license.