

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 8 2 8				PAGE (3) 1 OF 2		
TITLE (4) Reactor Trip From Loss Of Main Generator Stator Cooling Water																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 5	0 3	8 5	8 5	0 0 9	0 0 0	5	2 4	8 5					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)														
1		20.402(b)				20.406(c)				XX 50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
1 1 0 1 0		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME										TELEPHONE NUMBER						
Michael E. Frye, Compliance Section Engineer										AREA CODE 6 1 5 8 7 0 7 6 7 6 7						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												XX NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 3, 1985, at 1209 CST with unit 1 in cold shutdown (mode 5) and unit 2 operating at 100 percent reactor power, the unit 2 reactor tripped due to a main turbine trip. The main turbine trip was caused by a main generator trip which was due to a loss of stator cooling water.

All attendant equipment functioned properly, and there was no effect on the public health and safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Sequoyah, Unit 2	0 5 0 0 0 3 2 8	8 5	— 0 0	9 — 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Unit 2 tripped from 100 percent reactor power at 1209 CST on May 3, 1985, due to a main turbine trip which was caused by a main generator trip. The main generator was tripped due to a loss of both stator cooling water pumps. The unit stabilized in hot standby (mode 3) at 548 degrees F, and all attendant equipment functioned properly.

The 2B stator cooling water pump was being tested after maintenance for high vibration. The assistant shift engineer (ASE) started the 2B pump and placed the 2A stator cooling water pump in reserve. Plant test personnel, who were at the pump location to monitor the 2B pump vibration, noted electrical arcing and smoke on the 2A pump when the switching occurred and immediately called the control room to report the fire. The ASE did not see the fire on the 2A pump because the control panel is at a location remote to the pump. The control room sounded the fire alarm and announced the fire location over the plant intercom. The ASE heard the fire alarm sound but could not hear the fire location because of high background noise. He called the control room and was told the fire was on the stator cooling water pump. Because he had just started the 2B pump and could not see the pumps from his location, the ASE assumed the fire was on the 2B pump. He attempted to restart the 2A pump and stopped the 2B pump; however, the 2A pump was inoperable due to an electrical fault, and the action caused a total loss of stator cooling water which, after a 45-second time delay, caused the main generator to trip.

The electrical arc and smoke on the 2A pump did not result in a fire, and no plant fire fighting equipment was required. The condition was due to a faulty electrical connection and did not damage the motor. The vibration problem on the 2B pump was associated with the pump mounting bracket which was repaired and returned to normal service.

This event is considered to be isolated, and the probability of recurrence is extremely small; however, for the sake of good design practice, the relocation of the pump control station to a position adjacent to the pumps is being considered.

After the trip was cleared, it was discovered that reactor trip breaker B would not close because of a false train B intermediate range high flux trip signal. Plant maintenance personnel determined the problem was associated with the train B SSPS logic. A stray Termi point connector (a small metallic friction wire connector) was found lodged between the logic board pen for the operation above 10 percent power (P-10) block and a spare input pen. This combination generated a reactor trip signal whenever the unit was below P-10 and had no effect on the normally blocked operation above P-10.

The Termi point connector was removed and the wiring was inspected to ensure no loose wires were present. A full logic test was performed upon completion of maintenance to ensure proper operation for return to service. This event had no adverse effect on the safe shutdown of the reactor.

This was the fifth reactor trip on unit 2 in 1985 and is the first trip in the history files due to stator cooling water pumps. The event had no effect on the public health and safety.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
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May 24, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

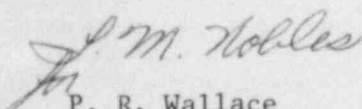
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET NO.
50-328 - FACILITY OPERATING LICENSE DPR-79 - REPORTABLE OCCURRENCE REPORT
SQRO-50-328/85009

The enclosed licensee event report provides details concerning a unit 2 reactor trip. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY


P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah

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