



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W.
ATLANTA, GEORGIA 30323

Report Nos.: 50-413/85-23 and 50-414/85-18

Licensee: Duke Power Company
422 South Church Street
Charlotte, NC 28242

Docket Nos.: 50-413 and 50-414

License Nos.: NPF-35 and CPPR-117

Facility Name: Catawba 1 and 2

Inspection Conducted: May 20 - 24, 1985

Inspector: *St. Schnelk for* 6/4/85
M. Thomas Date Signed

Approved by: *St. Schnelk for* 6/4/85
F. Jape, Section Chief Date Signed
Engineering Branch
Division of Reactor Safety

SUMMARY

Scope: This routine, unannounced inspection entailed 36 inspector-hours on site in the areas of preoperational test procedure review, preoperational test witnessing, and review of a Safety Evaluation Report (SER) open item.

Results: No violations or deviations were identified.

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REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *B. F. Caldwell, Station Services
- *J. W. Cox, Technical Services Superintendent
- *C. W. Graves, Jr., Operations Superintendent
- *C. L. Hartzell, Compliance Engineer
- *R. A. Jones, Test Engineer
- *C. S. Kelly, Construction Instrumentation/Electrical
- P. G. LeRoy, Licensing Engineer
- *E. B. Miller, Senior Quality Assurance (QA) Engineer
- *T. D. Mills, Construction Engineer - Electrical
- *G. Smith, Maintenance Superintendent
- *R. A. Throneburg, Construction Status and Control
- *E. G. Williams, Project QA Technician

Other licensee employees contacted included test coordinators, engineers, technicians, operators, and office personnel.

NRC Resident Inspectors

- *P. H. Skinner, Senior Resident Inspector, Operations
- *P. K. VanDoorn, Senior Resident Inspector, Construction

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on May 24, 1985, with those persons indicated in paragraph 1 above. The inspector described the areas inspected and discussed the inspection findings. No dissenting comments were received from the licensee. The licensee did not identify as proprietary any of the materials provided to or reviewed by the inspector during this inspection.

3. Licensee Action on Previous Enforcement Matters

This subject was not addressed in the inspection.

4. Unresolved Items

Unresolved items were not identified during the inspection.

5. Independent Inspection Effort (92706) - Units 1 and 2

The inspector conducted a general inspection of the Unit 2 auxiliary building, turbine building, reactor building, and the control room to observe ongoing construction and testing activities for compliance with NRC requirements and licensee commitments. In addition, the inspector reviewed the status of SER Open Item 9 concerning upgrading the pressurizer and steam generator power operated relief valves (PORVs) to safety related valves. The licensee has committed to upgrade the PORVs prior to startup following the first refueling outage for Unit 1 (License Condition 18) and prior to fuel load for Unit 2. In discussing this matter, licensee personnel stated that the modifications for Unit 1 are scheduled to be completed during the Unit 1 refueling outage. The pressurizer PORV modification for Unit 1 will be done under nuclear station modifications (NSMs) CN-10264 and CN-10436. The Unit 2 modifications will be performed under design change authorizations (DCAs) CN-2-M154 for the pressurizer PORVs and CN-2-M1527 for the steam generator PORVs. The Unit 2 pressurizer PORV modifications have been performed. Final QA acceptance and preoperational testing have not been completed yet. The steam generator modifications have not been completed. The inspector will continue to follow the progress of these modifications during subsequent inspections.

6. Preoperational Test Procedure Review (70304, 70311, and 70343) - Unit 2

The inspector reviewed the preoperational (preop) test procedures listed below to verify that they were consistent with applicable sections of the Final Safety Analysis Report (FSAR) Chapters 5, 6, 7, 9, 14, and 15, and Regulatory Guides 1.68 and 1.79. The following procedures were reviewed:

- TP/2/A/1200/02A, Residual Heat Removal System Functional Test
- TP/2/A/1200/03C, Safety Injection Pumps and Flow Adjustment Functional Test
- TP/2/A/1200/04, Containment Spray System Functional Test
- TP/2/A/1400/01, Nuclear Service Water System Functional Test

The procedures were reviewed to verify that the required reviews were performed, management approval was indicated, and acceptance criteria specified.

No violations or deviations were identified in the areas inspected.

7. Preoperational Test Witnessing (70315) - Unit 2

The inspector witnessed portions of preop tests TP/2/A/1200/02A (Sections 12.1, 12.3, 12.4, and 12.5) and TP/2/A/1200/03C (Section 12.14) which verified that interlocks for various system valves functioned properly. The tests were witnessed to verify the following:

- Latest revision of the test procedure was available and in use by test personnel.
- Test prerequisites were met.
- Proper plant systems were in service.
- The test was performed as required by the procedure.
- Test discrepancies were documented for review and resolution.
- Adequate coordination existed among personnel involved in the test.
- Temporary modifications such as jumpers were installed and tracked in accordance with administrative controls.

Section 12.3 of test TP/2/A/1200/02A was not completed successfully. Test discrepancies were written and the licensee is evaluating the problem. Additional sections of the above tests (which demonstrated pump performance criteria could be met for various conditions) were witnessed by the Resident Inspector and are documented in NRC Inspection Reports 50-414/85-05 and 50-414/85-06.

No violations or deviations were identified in the areas inspected.