

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
D. C. COOK NUCLEAR PLANT, UNIT 1

DOCKET NUMBER (2)

0 5 0 0 0 3 1 5 1 OF 0 1

PAGE (3)

TITLE (4)

TRAVEL OF HEAVY LOAD OVER SPENT FUEL

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)								
0	5	0	2	8	5	8	5	0	2	3	0	0	0	5	3	1	1	6
0	5	0	2	8	5	8	5	0	2	3	0	0	0	5	3	1	1	6

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)															
POWER LEVEL (10)	01010	20.402(b)		20.406(e)		50.73(a)(2)(iv)		73.71(b)									
		20.406(a)(1)(i)		50.38(a)(1)		50.73(a)(2)(v)		73.71(c)									
		20.406(a)(1)(ii)		50.38(a)(2)		50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
		20.406(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(vii)(A)											
		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(vii)(B)											
		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)											

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
J. D. ALLARD, MAINTENANCE SUPERINTENDENT	AREA CODE 616 465-5901

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO		0	7	15

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON MAY 2, 1985 WITH THE UNIT 1 REACTOR COOLANT SYSTEM IN MODE 6 (REFUELING) AND UNIT 2 REACTOR IN MODE 1 (POWER OPERATION) AT 100% THERMAL POWER, DISCUSSIONS WERE BEING HELD REGARDING AUXILIARY BUILDING CRANE (IEEE/CRN) TRAVEL LIMITS. IT BECAME APPARENT THAT THE DESIGNATION OF THE AUXILIARY BUILDING CRANE LOAD BLOCK AS A "HEAVY LOAD" (UNDER NUREG:0612) WAS NOT GENERALLY KNOWN. CONSEQUENTLY, MOVEMENT OF THE LOAD BLOCK OVER RACKS CONTAINING FUEL OCCURRED ON AT LEAST ONE OCCASION WHEN THE CRANE WAS USED AS A WORK PLATFORM TO FACILITATE REPAIRS TO AUXILIARY BUILDING HIGH BAY LIGHTS IN AUGUST, 1984. NO LOADS OTHER THAN THE LOAD BLOCK WERE MOVED. THIS EVENT MAY NOT BE IN COMPLIANCE WITH THE REQUIREMENTS OF TECHNICAL SPECIFICATION 3.9.7.

AN EVALUATION IS CURRENTLY BEING PERFORMED TO DETERMINE IF, IN FACT, THE LOAD BLOCK POSTULATED LOAD DROP IMPACT ENERGY EXCEEDS THE LIMIT OF TECHNICAL SPECIFICATION 3.9.7.

THE DEFINITION OF THE LOAD BLOCK AS BEING A HEAVY LOAD WAS NOT SPECIFICALLY EMPHASIZED IN PLANT PROCEDURES. AS A PRECAUTIONARY MEASURE, CAUTIONS ARE BEING ADDED TO APPLICABLE PLANT PROCEDURES AND WILL BE IN PLACE BY 6/15/85. ADDITIONALLY, A MEMO HAS BEEN ISSUED TO ALL PLANT DEPARTMENT HEADS AND MAINTENANCE DEPARTMENT EMPLOYEES WHICH CLEARLY DESCRIBES THE LOAD BLOCK "HEAVY LOAD" DESIGNATION.

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PDR ADOCK 05000315
S PDR

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LICENSEE EVENT REPORT REVIEW/EVALUATION FORMCONDITION REPORT NO.: 12-05-85-882
RESPONSIBLE DEPARTMENT: STALER NO.: _____
REVISION NO.: 0
UNIT NO.: 1

- X 1. A brief abstract describing the major occurrences during the event, including: a) all component or system failures that contributed to the event and b) significant corrective action taken or planned to prevent recurrence.
- X 2. A clear, specific, narrative description within the text of what occurred so that knowledgeable readers conversant with the design of commercial nuclear power plants, but not familiar with the details of a particular plant, can understand the complete event. The narrative description must include the following specific information as appropriate for the particular event:
- X (A) Plant operating conditions before the event.
- NA (B) Status of structures, components, or systems that were inoperable at the start of the event and that contributed to the event.
- X (C) Dates and approximate times of occurrences.
- NA (D) The cause of each component or system failure or personnel error, if known.
- NA (E) The failure mode, mechanism, and effect of each failed component, if known.
- X (F) The Energy Industry Identification System component function identifier and system name of each component or system referred to in the LER. The Energy Industry Identification System is defined in: IEEE Std. 803-1983 (May 16, 1983) Recommended Practices for Unique Identification Plants and Related Facilities - Principles and Definition.
- NA (G) For failure of components with multiple functions, include a list of systems or secondary functions that were also affected.
- NA (H) For failure that rendered a train of a safety system inoperable, an estimate of the elapsed time from the discovery of the failure until the train was returned to service.
- X (I) The method of discovery of each component or system failure or procedural error.
- NA (J) (1) Operator actions that affected the course of the event, including operator error*, procedural deficiencies, or both, that contributed to the event.
- NA (2) For each personnel error, the licensee shall discuss:
- NA (a) Whether the error was a cognitive error (e.g., failure to recognize the actual plant condition, failure to realize which systems should be functioning, failure to recognize the true nature of the event or a procedural error;
- NA (b) Whether the error was contrary to an approved procedure, was a direct result of an error in an unapproved procedure, or was associated with an activity or task that was not covered by an approved procedure;
- NA (c) Any unusual characteristics of the work location (e.g., heat, noise) that directly contributed to the error; and
- NA (d) The type of personnel involved (i.e., contractor personnel, utility-licensed operator, utility non-licensed operator, other utility personnel).
- NA (K) Automatically and manually initiated safety system responses.
- NA (L) The manufacturer and model number (or other identification) of each component that failed during the event.
- X 3. An assessment of the safety consequences and implications of the event. This assessment must include the availability of other systems or components that could have performed the same function as the components and systems that failed during the event.
- X 4. A description of any corrective/preventive actions planned as a result of the event, including those to reduce the probability of similar events occurring in the future.
- X 5. Reference to any previous similar events at the same plant that are known to the licensee.
- X 6. The name and telephone number of a person within the licensee's organization who is knowledgeable about the event and can provide additional information concerning the event and the plant's characteristics.

NOTE: Item Nos. 1 through 6 are written as described in NUREG-1022.

X indicates that the item was included in the LER.N/A indicates that the item is not applicable to the LER.



INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
(616) 465-5901

May 31, 1985

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Operating License DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10CFR:0.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 85-023-85-0

Sincerely,

W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
J.G. Keppler, RO:III
M.P. Alexich
R.F. Kroeger
H.B. Brugger
R.W. Jurgensen
NRC Resident Inspector
R.C. Callen, MPSC
G. Charnoff, Esq.
J.M. Hennigan
INPO
PNSRC
J.F. Stietzel
A.A. Blind
Dottie Sherman, ANI Library

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11