

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 5 1 OF 0 1					PAGE (3) 1 OF 0 1	
TITLE (4) Primary Containment Group 6 Isolation, Reactor Building Ventilation Isolation, and Automatic Start of Reactor Building Standby Gas Treatment System Train 1B																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 5	0 2	8 5	8 5	0 2 2	0 0	0 5	3 0	8 5					0 5 0 0 0			
OPERATING MODE (9) 5			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)													
POWER LEVEL (10) 0 0 0			20.402(b)				20.406(c)				X 50.73(a)(2)(iv)				73.71(b)	
			20.406(a)(1)(i)				50.36(e)(1)				50.73(a)(2)(v)				73.71(c)	
			20.406(a)(1)(ii)				50.36(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 386A)	
			20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
			20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
			20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)																
NAME M. J. Pastva, Jr., Regulatory Technician										TELEPHONE NUMBER AREA CODE 9 1 9 4 5 7 - 2 3 1 5						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS						
X	B, H	3 0	A, I 6 0	Yes												
SUPPLEMENTAL REPORT EXPECTED (14)																
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a Unit 1 refuel/maintenance outage, on 5-02-85, at 1407, a primary containment Group 6 isolation occurred, the Reactor Building ventilation system automatically isolated, and train 1B of the Reactor Building Standby Gas Treatment (SBGT) System automatically started. A Reactor Building fire watch had observed smoke emanating from SBGT train 1A and reported the discovery to the Control Room. Operations personnel assessed the problem and deenergized the power supply to Trouble Start Control Relay CR-4 of SBGT train 1A, thereby resulting in the event. CR-4, which is in the circuitry to the unit SBGT trains' high temperature Control Room annunciator, had overheated and caught fire. The fire was reported out at 1418.

SBGT train 1B was secured and returned to standby, and the isolation signals were reset. The involved smoke did not affect operability of SBGT train 1B. On 5-07-85, CR-4 (Allen-Bradley Part No. 696-521-39) to SBGT 1A was replaced. The failure of CR-4 is attributed to relay coil insulation breakdown resulting from indeterminate causes.

The CR-4 relays in SBGT 1B and Unit 2 SBGTs 2A and 2B were visually inspected for signs of overheating or physical degradation with no problems found.

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Carolina Power & Light Company

Brunswick Steam Electric Plant  
P. O. Box 10429  
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May 30, 1985

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BRUNSWICK STEAM ELECTRIC PLANT UNIT 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
LICENSEE EVENT REPORT 1-85-022

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

*C. R. Dietz for*

C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

MJP/clh

Enclosure

cc: Dr. J. N. Grace

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