

Enclosure 1

NOTICE OF VIOLATION

GPU Nuclear Corporation
Three Mile Island Nuclear Station, Unit 1

Docket No. 50-289
License No. DPR-50

During an NRC inspection, conducted June 17, 1996 - August 3, 1996, a violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," (60 FR 34381; June 30, 1995), the violation is listed below:

10 CFR 50, Appendix B, Criteria III, Design Control, states that "Measures shall be established to assure that applicable regulatory requirements and the design basis... are correctly translated into specifications..." and "The design control measure shall provide for verifying or checking the adequacy of design...".

The Licensee's "GPU Nuclear Operational Quality Assurance Plan," section 4 "Design Control," Paragraph 4.2.9 states, in part: Design verification procedures shall be established.....

Procedure 500C ADM-7311.02 (EP-009) "Design Verification," section 4.3 "Verification Process," Paragraph 4.3.1 states: Design input shall be verified by comparing the documented input with the source reference, and by checking the validity for the intended use. All assumptions shall be evaluated and verified to be based on sound engineering principles and practices. Paragraph 4.3.8 states: In addition to the responsibilities of a Design Verification Engineer, the Lead Design Verification Engineer is responsible for the overall design verification and for assembling the design verification package. This includes interfacing with the Design Engineers and Managers to coordinate the overall design verification effort. Section 4.9 "Documentation of Design Verification," Paragraph 4.9.7 states, in part: The verification package shall be completed by the Lead Design Verification Engineer to document the completed design verification.

Contrary to the above, the inspectors could not confirm "valve factor" inputs to the calculations which establish design basis functionality of motor operated valves in safety related applications. The licensee did not verify or document, in accordance with procedure 5000-ADM-7311.02 (EP-009) "Design Verification," the valve factors for Motor Operated Valves RC-V-2 (Power Operated Block Valve) and FW-V-92A&B (Startup Feedwater Block Valves).

This is a Severity Level IV violation (Supplement IV).

Pursuant to the provisions of 10 CFR 2.201, GPU Nuclear Corporation is hereby required to submit a written statement or explanation to the U. S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region I, and a copy to the NRC Resident Inspector at the Facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of

Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without reaction. However, if you find it necessary to include such information, you should clearly indicate the specific information that you desire not to be placed in the PDR, and provide the legal basis to support your request for withholding the information from the public.

Dated at King of Prussia, Pennsylvania
this 13 day of September, 1996