

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1)										DOCKET NUMBER (2)				PAGE (3)			
Dresden Nuclear Power Station, Unit 3										0   5   0   0   0   2   4   9				1   OF   0   2			

TITLE (4)

Local Leak Rate Test Failure of Feedwater Valve 3-220-62A

EVENT DATE (6)			LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)														
									N/A					0   5   0   0   0														
0	4	2	9	8	5	8	5	0	1	2	0	0	0	5	2	1	8	5	N/A					0   5   0   0   0				

OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)									
POWER LEVEL (10)	0   0   0		20.402(b)		20.406(e)		50.73(a)(2)(iv)		73.71(b)			
			20.405(a)(1)(i)		50.36(e)(1)		50.73(a)(2)(v)		73.71(e)			
			20.405(a)(1)(ii)		50.36(a)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 365A)			
			20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Brian McCabe (X-550)	AREA CODE 815 942-2920

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (12)											
CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS	
X	S I	S E A I L	C 6 6 5	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

**ABSTRACT** (Limit to 1400 spaces i.e. approximately fifteen single-space typewritten lines) (16)

During the Unit 3 short outage, while performing DTS 1600-1 (Local Leak Rate Testing of Primary Containment), feedwater check valve 3-220-62A was observed to leak 587.04 SCFH. This leakage, added to the previous "as left" leakage from the 1983-84 Unit 3 refuel outage, results in a total leakage of 812.74 SCFH. This exceeds the Technical Specifications limit for Type B and C leak rate testing which is 493.116 SCFH.

An inspection of the valve showed that the pressure seal ring was worn. This seal ring was replaced and an "as left" LLRT was performed. This test resulted in a leakage which brought the total Type B and C leakage to below Technical Specification limits. A previous occurrence of a failure of Type B and C leak testing was reported by reportable occurrence 84-23 on Docket #050237. Safety significance was minimal since the calculated "through leakage" for the primary containment was always well below the Technical Specifications limits.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Dresden Nuclear Power Station, Unit 3	0 5 0 0 0 2 4 9	8 5	— 0 1 2	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

During the Unit 3 short outage, while performing DTS 1600-1 (Local Leak Rate Testing of Primary Containment), feedwater check valve 3-220-62A was observed to leak 587.04 SCFH. This leakage, added to the previous "as left" leakage from the 1983-84 Unit 3 refuel outage, results in a total leakage of 812.74 SCFH. This exceeds the Technical Specifications limit for Type B and C leak rate testing which is 493.116 SCFH.

Prior to the Unit 3 short outage, the Mechanical Maintenance Department noticed excessive leakage from the 3-220-62A valve. Before work was performed on the valve, an LLRT was performed. This test resulted in a leakage of 587.04 SCFH. Using the conservative "Maximum Pathway" method for determining leakage, this leakage added to the previous "as left" leakage from the 1983-84 Unit 3 refuel outage exceeds the Technical Specifications limit for Type B and C leak rate testing. An inspection of the valve showed that the pressure seal ring was worn. This seal ring was replaced and an "as left" LLRT was performed. This test resulted in a leakage which brought the total Type B and C leakage to below Technical Specification limits. Safety significance was minimal since the calculated "through leakage" for the primary containment was always well below the Technical Specifications limit of 616.39 SCFH. A previous occurrence of a failure of Type B and C leak testing was reported by Reportable Occurrence 84-23 on Docket #050237.



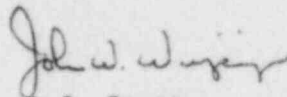
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Dresden Nuclear Power Station  
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Morris, Illinois 60450  
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May 21, 1985

DJS Ltr #85-565

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Washington, D.C. 20555

Licensee Event Report #85-012-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(i)(B).

*for*   
D.J. Scott  
Station Manager  
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III  
File/NRC  
File/Numerical

*IE22*  
*11*