

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Nine Mile Point Unit #1										DOCKET NUMBER (2) 0 5 0 0 0 2 2 0 1 0 0 2				PAGE (3) 1 of 0 2	
TITLE (4) Initiation of Reactor Building Emergency Ventilation Due to Power Supply Short															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)		
0 4	2 4	8 5	8 5	0 0 7	0 0	0 5	2 4	8 5					0 5 0 0 0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)													
N		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)	
POWER LEVEL (10)		20.406(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)	
1 0 0		20.406(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)															
NAME Robert Randall, Supervisor, Technical Support										TELEPHONE NUMBER 3 1 5 3 4 9 - 2 4 4 5					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE
X	I	L	C	O	N	A	3	8	10	Y					
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

During normal operation on April 24, 1985 the automatic initiation of the Reactor Building Emergency Ventilation System occurred. The initiation was the result of a technician shorting a wire in the cable connector to the sensor for the Emergency Condenser Ventilation Monitor No. 121.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Nine Mile Point Unit #1	0500022085	—	007	—	000	2 OF	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

TEXT

A maintenance work request had been issued for sensor convertor cables for the No. 121 Emergency Condenser Vent Radiation Monitor. During reassembly of the cable connector, a technician shorted a wire to ground causing the external trip unit and internal power supply fuses to blow. This resulted in the Channel 11 24 volt power supply being shorted out causing off-scale radiation level indications to the No. 11 Reactor Building Ventilation Duct Radiation Monitor. This monitor subsequently tripped initiating the actuation of the Reactor Building Emergency Ventilation System. This power supply supplies 24 volt DC power for the following radiation monitors: 1) Containment Spray 111 Hx Effluent, 2) Containment Spray 112 Hx Effluent, 3) Emergency Condenser Vent 111, 4) Emergency Condenser Vent 121, 5) Reactor Building Ventilation 11, and 6) Constant Air Monitors for the Reactor Building, Turbine Building, Waste Building and Drywell.

The external 24 volt fuse and the 24 volt power supply were replaced. Although the 24 volt power supply was not damaged, time constraints necessitated replacing the power supply. The faulty cable connector was re-repaired per the initial work request. The No. 11 radiation monitoring channel supplied by the shorted 24 volt power supply was satisfactorily returned to service.

ASSESSMENT OF POTENTIAL SAFETY CONSEQUENCES

The initiation of the Reactor Building Emergency Ventilation System is a conservative mode and thus posed no threat to the safety of the plant. The No. 12 radiation monitoring channel provides redundant monitoring protection and would have been able to initiate emergency ventilation if required. The time constraint for additional radiation monitoring protection was not exceeded thus requiring no additional action per technical specifications.

There were no adverse consequences from this event and the potential consequences are within the design basis of the plant.

CORRECTIVE ACTION

The damage caused by the shorting of a wire in the sensor convertor cable connector for the No. 121 Emergency Condenser Ventilation Radiation Monitor was repaired. An external 24 volt fuse and internal 24 volt power supply fuse blew. The external fuse was replaced and due to time constraints the entire power supply was replaced. The faulty cable connector was re-repaired per the initial work request. The Channel 11 radiation monitoring channel supplied by the shorted 24 volt power supply was then satisfactorily returned to service.

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

May 24, 1985

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555RE: Docket No. 50-220
LER 85-07

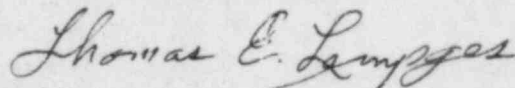
Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following
Licensee Event Report:

LER 85-07 Which is being submitted in accordance with
10 CFR 50.73 (a)(2)(iv), "Any event or condition
that resulted in manual or automatic actuation of
any Engineered Safety Feature (ESF), including
the Reactor Protection System (RPS). However,
actuation of an ESF, including the RPS, that resulted
from and was part of the preplanned sequence during
testing or reactor operation need not be reported."

This report was completed in the format designated in NUREG-1022,
dated September 1983.

Very truly yours,

Thomas E. Lempsges
Vice President
Nuclear Generation

TEL/lo

cc: Dr. Thomas E. Murley
Regional Administrator
attachmentsLE22
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