

MAY 22 1985

In Reply Refer To:  
Docket: 50-267/84-32

Public Service Company of Colorado  
ATTN: O. R. Lee, Vice President  
Electric Production  
P. O. Box 840  
Denver, Colorado 80201

Dear Mr. Lee:

This refers to the Management Conference conducted in the Region IV offices on November 14, 1984, to review the major conclusions of NRC's October 1984 Assessment Report, to discuss operations of the Fort St. Vrain facility and to review outstanding licensing actions. Since this conference there have been a number of technical meetings with your staff and you have submitted a substantial amount of information pertaining to the matters that were discussed. We have also carried out a number of related inspections.

Based on a detailed review of records related to issues raised in the Assessment Report and on follow-up inspection effort, we have concluded that no violations of NRC requirements were identified therein. However, there are two matters which must be brought to your attention.

As we discussed with you and members of your staff on several different occasions, it is possible to come to an ambiguous interpretation of 10 CFR 50.72 concerning whether the failure of six control rods to automatically insert during the scram event which occurred on June 23, 1984, should have been reported at the time of the initial report to the NRC Operations Center.

You should emphasize to your operations staff that it is the intent of this regulation that any known condition involving potentially serious degradation of a component or system required for plant safety be reported at the time an event notification is made pursuant to 10 CFR 50.72. Likewise, your training programs and plant procedures should be clear on this point.

Section 4.2.5.6 (page 4-8) of the Assessment Report discusses your program for verification of the correct performance of operating activities. (TMI Action Plan Requirements, NUREG-0737, item 1.C.6.) Your position on implementation of this item was apparently accepted without sufficient consideration to the need for double verification of system or equipment status, particularly in connection with maintenance activities. You should review your procedures

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5/21/85

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with this in mind and assure that you implement a system of "double verification" to the maximum extend practicable in view of the design of your plant. Your response to this item should be provided within 120 days of the date of this letter. If you have any questions concerning this letter, we will be pleased to discuss this with you.

Sincerely,

Original Signed By

E. H. Johnson

E. H. Johnson, Chief  
Reactor Project Branch 1

Enclosure:

Appendix - NRC Inspection Report  
50-267/84-32

cc:

Mr. D. W. Warembourg, Manager  
Nuclear Engineering Division  
Public Service Company of Colorado  
P. O. Box 840  
Denver, Colorado 80201

Mr. David Alberstein, 14/159A  
GA Technologies, Inc.  
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San Diego, California 92138

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Public Service Company Building  
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Denver, Colorado 80202

Chairman, Board of County Comm.  
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Greeley, Colorado 80631

Regional Representative  
Radiation Programs  
Environmental Protection Agency  
1860 Lincoln Street  
Denver, Colorado 80203

(cont. on next page)

Public Service Company of Colorado

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Mr. H. L. Brey, Manager  
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Denver, Colorado 80201

J. W. Gahm, Manager, Nuclear  
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16805 WCR 19 $\frac{1}{2}$   
Platteville, Colorado 80651

L. Singleton, Manager, Quality  
Assurance Division  
(same address)

Colorado Radiation Control Program Director

bcc distrib. by RIV:

- \*RPB1
- \*Resident Inspector
- \*Section Chief (RPB1/SPES)
- \*P. Wagner, RPB1
- \*D. Powers, RPB1
- \*EP&RPB
- \*MIS System
- \*RSTS Operator
- \*RIV File

- R. P. Denise, DRSP
- R. D. Martin, RA
- E. Haycraft, LA
- J. Miller, ORB3
- \*D. Weiss, LFMB (AR-2015)

T. F. Westerman, EO

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