



Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier

Vice President Nuclear Operations
and Station Director

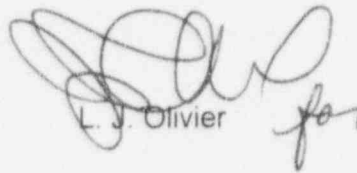
September 12, 1996
BECO Ltr. #96-083

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

AUGUST 1996 MONTHLY REPORT

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report, please contact me directly.



L. J. Olivier

RLC/dmc/9458

Attachment

cc: Mr. Hubert Miller
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO.	50-293
NAME:	Pilgrim
COMPLETED BY:	R. L. Cannon
TELEPHONE:	(508) 830-8321
REPORT MONTH	August 1996

OPERATING STATUS

NOTES

- | | | |
|-----|---|-------------|
| 1. | Unit Name | Pilgrim I |
| 2. | Reporting Period | August 1996 |
| 3. | Licensed Thermal Power (MWt) | 1998 |
| 4. | Nameplate Rating (Gross MWe) | 678 |
| 5. | Design Electrical Rating (Net MWe) | 655 |
| 6. | Maximum Dependable Capacity (Gross MWe) | 696 |
| 7. | Maximum Dependable Capacity (Net MWe) | 670 |
| 8. | If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons: | |
| | No Changes | |
| 9. | Power Level To Which Restricted, If Any (Net MWe): <u>None</u> | |
| 10. | Reasons For Restrictions, If Any: <u>N/A</u> | |

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	744.0	5855.0	208007.0
12. Hours Reactor Critical	744.0	5740.5	132006.6
13. Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14. Hours Generator On-Line	744.0	5721.9	127545.8
15. Hours Unit Reserve Shutdown	0.0	0.0	0.0
16. Gross Thermal Energy Generated(MWH)	1411264.0	11115746.0	227417916.0
17. Gross Electrical Energy Generated(MWH)	478260.0	3801720.0	77079044.0
18. Net Electrical Energy Generated(MWH)	459908.0	3662268.0	74105890.0
19. Unit Service Factor	100.0	97.7	61.3
20. Unit Availability Factor	100.0	97.7	61.3
21. Unit Capacity Factor (Using MDC Net)	92.3	93.4	53.2
22. Unit Capacity Factor (Using DER Net)	94.4	95.5	54.4
23. Unit Forced Outage Rate	0.0	0.0	11.5

24. Shutdowns Scheduled Over Next 6 Months
(Type, Date, and Duration of Each) -

Refueling outage scheduled for February 1, 1997. Expected duration is 42 days.

25. If Shutdown at End of Report Period,
Estimated Date of Startup -

UNIT OPERATING

AVERAGE DAILY UNIT POWER LEVEL

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DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	659	17	643
2	656	18	654
3	658	19	652
4	657	20	651
5	656	21	656
6	652	22	654
7	651	23	628
8	652	24	384
9	653	25	413
10	653	26	415
11	650	27	465
12	648	28	626
13	649	29	651
14	649	30	650
15	649	31	579
16	650		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATIONAL SUMMARY

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REPORT MONTH	<u>August 1996</u>

The plant entered August operating at 100% core thermal power (CTP) where it was essentially maintained until 9/23. On 8/23, at 2046 hours, the plant commenced reducing power to approximately 60% CTP to facilitate scheduled power suppression testing of the nuclear fuel and perform selective maintenance in the condenser bay. On 8/28, at 1200 hours, the plant attained 100% CTP where it was maintained until 8/31. On 8/31, at 0815 hours, the plant reduced power to approximately 90% CTP to perform control rod exercising. At 0900 hours, power was further reduced to approximately 70% CTP to perform maintenance on control rod 02-27. The plant attained 100% CTP at 2350 hours ending August at full power.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF AUGUST 1996

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

DOCKET NO.	<u>50-293</u>
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REPORT MONTH	<u>August 1996</u>

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: February 1, 1997.
3. Scheduled date for restart following next refueling: March 14, 1997.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1765 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
 - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

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SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
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No major safety related corrective maintenance was completed during this reporting period.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
18	08/23/96	S	0.0	B	N/A	N/A	N/A	N/A	Planned down power to conduct scheduled power suppression testing of the nuclear fuel and perform selective maintenance.
19	08/31/96	S	0.0	B	N/A	N/A	N/A	N/A	Planned powered reduction to perform maintenance on control rod 02-27.

1

F-Forced
S-Sched

2

A-Equip Failure
 B-Main or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training
 & License Examination
 F-Admin
 G-Operator Error
 H-Other

3

1-Manual
 2-Manual Scram
 3-Auto Scram
 4-Continued
 5-Reduced Load
 9-Other

4&5

Exhibit F & H
 Instructions for
 Preparations of
 Data Entry Sheet
 Licensee Event Report
 (LER) File (NUREG-1022)