

GPU NUCLEAR CORPORATION  
OYSTER CREEK NUCLEAR GENERATING STATION

---

Operating License No. DPR-16  
Docket No. 50-219  
Technical Specification Change Request No. 203

---

This Technical Specification Change Request is submitted in support of the Licensee's request to change the Appendix A Technical Specifications to Operating License No. DPR-16 for Oyster Creek Nuclear Generating Station. As a part of this request, the proposed replacement pages for Appendix A are also submitted.

GPU Nuclear Corporation

Michael B. Roche

Michael B. Roche  
Vice President and Director  
Oyster Creek

Sworn and Subscribed to before me this 10th day of October, 1996.

Geraldine E. Levin

A Notary Public of New Jersey

GERALDINE E. LEVIN  
NOTARY PUBLIC OF NEW JERSEY  
My Commission Expires 6-8-2000

9610220180 961010  
PDR ADOCK 05000219  
P PDR

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the matter )  
GPU Nuclear Corporation )

Docket No. 50-219

CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 203 for Oyster Creek Nuclear Generating Station Technical Specifications, filed with the U.S. Nuclear Regulatory Commission on 10/10/ , 1996, has this day of 10/10/ , 1996, been served on the Major of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable John Parker  
Mayor of Lacey Township  
818 West Lacey Road  
Forked River, NJ 08731

By Michael B. Roche  
Michael B. Roche  
Vice President and Director  
Oyster Creek



GPU Nuclear, Inc.  
U.S. Route #9 South  
Post Office Box 388  
Forked River, NJ 08731-0388  
Tel 609-971-4000

October 10, 1996  
6730-96-2157

The Honorable John Parker  
Mayor of Lacey Township  
818 West Lacey Road  
Forked River, New Jersey 08731

Dear Mayor Parker:

Subject: Oyster Creek Nuclear Generating Station  
Operating License No. DPR-16  
Technical Specification Change Request No. 203

Enclosed herewith is one copy of Technical Specification Change Request No. 203 for the Oyster Creek Nuclear Generating Station Operating License.

This document was filed with the United States Nuclear Regulatory Commission on October 10, 1996.

Very truly yours,

A handwritten signature in cursive script that reads "Michael B. Roche".

Michael B. Roche  
Vice President and Director  
Oyster Creek

MBR/gmg  
Attachment



GPU Nuclear, Inc.  
U.S. Route #9 South  
Post Office Box 388  
Forked River, NJ 08731-0388  
Tel 609-971-4000

October 10, 1996  
6730-96-2157

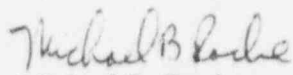
Mr. Kent Tosch, Chief  
Bureau of Nuclear Engineering  
Department of Environmental Protection  
CN 411  
Trenton, NJ 08625

Dear Mr. Tosch:

Subject: Oyster Creek Nuclear Generating Station  
Operating License No. DPR-16  
Technical Specification Change Request No. 203

Pursuant to 10 CFR 50.91(b)(1), please find enclosed a copy of the subject document which was filed with the United States Nuclear Regulatory Commission on *October 10*, 1996.

Very truly yours,

  
Michael B. Roche  
Vice President and Director  
Oyster Creek

MBR/gmg  
Attachment

**Oyster Creek Nuclear Generating Station (OCNGS)**  
**Operating License No. DPR-16**  
**Docket No. 50-219**  
**Technical Specification Change Request No. 203**

1.0. PROPOSED TECHNICAL SPECIFICATION CHANGE REQUEST (TSCR)

GPUN requests that the following pages of the OCNGS Technical Specifications (Tech. Specs.) be replaced as indicated below:

Replace Pages: 4.1-4, 4.1-5, 4.1-7, 4.3-2, 4.4-1, and, 4.5-4. These pages are attached to this change request.

2.0. DESCRIPTION OF CHANGES

License Amendment No. 144 (dated 1/10/91) revised the Appendix A - Technical Specifications to implement a 24 month plant surveillance cycle. Part of the change revised certain TS surveillance intervals to specify a 20 month interval following the redefinition of the "refueling outage interval" to 24 months (TS 1.12). The surveillances which had not yet been fully evaluated for extension to 24 months were thus limited to a 20 month frequency (the refueling interval definition existing prior to License Amendment No. 144). The purpose of TSCR 203 is to change the surveillance interval for the next group of surveillance frequency evaluations recently completed.

3.0. DISCUSSION OF THE REASONS FOR CHANGE

The following discussion provides justification for extending the interval for the surveillances in accordance with the guidance (where appropriate) contained in NRC Generic Letter 91-04, Enclosure 2. The TS instrumentation surveillances for extension are:

<u>TS Table</u>	<u>Item</u>	<u>Description</u>
4.1.1	18	Condenser Low Vacuum
4.1.1	20	High Temperature Main Steamline Tunnel
4.1.1	25	Recirculation Flow
4.3.H	3	Reactor Coolant System Leakage

**Oyster Creek Nuclear Generating Station (OCNGS)**  
**Operating License No. DPR-16**  
**Docket No. 50-219**  
**Technical Specification Change Request No. 203**

In addition to the above instrumentation surveillances to be extended by this request, we hereby also request that the following equipment tests/operability checks be extended to conform with the 24 month refueling interval:

TS	Item	Description
4.1.2	13	Containment Vent & Purge Isolation
4.4.B	1	EMRV Valve Operability
4.5.F	5.b.(4)	Drywell to Torus Leakrate Test

**SAFETY EVALUATION JUSTIFYING CHANGES:**

All of the following instruments and equipment have had an evaluation performed to determine the effect of surveillance frequency change on their performance requirements. For all instruments, an error analysis was performed to calculate the maximum possible deviation between surveillance intervals. Except where noted below, a statistical analysis (linear regression and t-distribution for confidence level) using historical calibration data was used to predict the deviation, for 30 months (24 months +/- 25%) with a 95% confidence level. In the case of equipment testing, the historical performance during past surveillances was evaluated for potential failures. Based on the results of the evaluations performed, changing the surveillance frequency from 20 months to 24 months/refueling outage interval is acceptable.

Condenser Low Vacuum

TS Table 4.1.1, Item 18, requires the calibration and test of the condenser low vacuum instrument channel once every 20 months. The channel consists of (4) four limit switches that monitor condenser vacuum and initiate a trip signal to the Reactor Protection System at a low vacuum. The Technical Specification limit for this setpoint is  $\geq 20$  inches Hg. Based on the statistical analysis of historical calibration data it is concluded that changing the calibration interval to 24 months (+/- 25%, or 30 months max.) will not impact upon existing as-found limits or the Technical Specification limit.

**Oyster Creek Nuclear Generating Station (OCNGS)**  
**Operating License No. DPR-16**  
**Docket No. 50-219**  
**Technical Specification Change Request No. 203**

High Temperature Main Steamline Tunnel

TS Table 4.1.1, Item 20, requires the calibration of the main steamline tunnel instrumentation channel once every 20 months. Two instrument channels provide high temperature monitoring in the main steamline tunnel to detect a main steamline break in this area and close the MSIVs. Each channel consists of two subchannels having four temperature sensors per subchannel. The circuit is arranged with a one out of two taken twice logic. The Technical Specification limit is a Trip Setting of  $\leq$  Ambient temperature at Power + 50F. The evaluations performed show that the deviation of instrument setpoint due to drift over a 30 month period is very small indicating that temperature sensors will perform their required function of closing the MSIVs upon detection of a steam line break.

Recirculation Loop Flow

TS Table 4.1.1, Item 25, requires the calibration of the recirculation flow monitoring system by application of a test pressure once every 20 months. The recirculation flow monitoring system consists of flow transmitters, summation logic, and the APRM flow units. The flow transmitters and control room electronics were replaced in refueling outage 15R; so no historical data is available. Therefore an instrument loop error calculation was performed based upon vendor specifications for the electronics (as provided for in GL 91-04). The results of the calculation show that with existing setpoints and the 30 month total instrument loop error considered all system functions will perform acceptably and are not adversely impacted by the extended calibration interval.

Reactor Coolant System Leakage

TS 4.3 H., Item 3, requires a channel calibration of the primary containment sump and equipment drain tank flow integrators once every 18 months. The two flow integrator channels consist of flow transmitters, signal converters, and combined square root integrator modules. In addition, each channel has a totalizer and indicator. The drywell floor sump is used to determine unidentified leak rates and the equipment drain tank flow provides the identified primary leak rates. The Technical Specification 3.3.D.1 limit is 5 gpm for unidentified leakage, and 25 gpm for total leakage. Statistical analysis for the transmitter and flow integrator predicted the deviation for a 30 month surveillance interval as a significantly lower value compared to the acceptance criteria. Therefore, it is concluded that the instrument loop will perform its intended function for the extended surveillance interval.

**Oyster Creek Nuclear Generating Station (OCNGS)**  
**Operating License No. DPR-16**  
**Docket No. 50-219**  
**Technical Specification Change Request No. 203**

Containment Vent & Purge Isolation

TS Table 4.1.2, Item 1 requires a trip system test of containment vent and purge isolation once every 20 months. The sensors which trip the containment purge and isolation system valves are calibrated in accordance with TS Table 4.1.1, Item 29. The trip system logic is an electrical circuit consisting of test switches and solenoids. Test results for a five year period were reviewed and during this period there were no failures of the trip system to initiate and perform its intended function. Based upon the history of satisfactory test results, it is concluded that extending the surveillance interval to 24 mos. (+/- 25%, or 30 months) will have no adverse impact on the required function of the valves and therefore the extended interval is acceptable.

EMRV Valve Operability

TS 4.4.B., Item 1, requires a demonstration of the Automatic Depressurization System (ADS) valve operability at system operating pressure prior to exceeding 5% power following a refueling outage and on an interval not to exceed 20 months. A review of the valve operability tests performed over a five year period shows that there were no failures of a valve to operate when called upon using 20 month interval testing. Based upon its performance history an extension of the periodic surveillance interval to 24 months (+/- 25%, or 30 months) should not compromise the ability of the valves to perform their intended safety function.

Drywell to Torus Leakrate Test

TS 4.5.F., Item 5.B.4, requires a drywell-to-torus leak rate test on a periodic interval not to exceed 20 months. The Torus to Drywell vacuum breaker valves perform the following functions: (a.) open to the full open position with a 0.5 psid force acting on face of the valve disk to assure that the 2.0 psid pressure between the drywell and external environment is not exceeded; (b.) limits negative pressure differentials on the drywell; and, (c.) provides for primary containment integrity. A review of Leakrate Test history from 1977 to present was performed and the results indicate that there was no degradation over this time period. In addition, a quarterly leak rate test has been performed at power since March 1990 pursuant to TS 3.5.A.5 and the acceptance criteria have been met repeatedly. These valves are also cycled on a monthly basis to ensure operability as required by TS 4.5.F. Item 5.a. Based on the performance history above and the additional Technical Specification requirements, it is concluded that the extension of this test from 20 months to a refueling basis interval of 24 mos. (+/- 25%, or 30 months) is acceptable.

**Oyster Creek Nuclear Generating Station (OCNGS)**  
**Operating License No. DPR-16**  
**Docket No. 50-219**  
**Technical Specification Change Request No. 203**

4.0 NO SIGNIFICANT HAZARDS CONSIDERATIONS:

GPUN has determined that this Technical Specification Change Request involves no significant hazards consideration as defined by NRC in 10 CFR 50.92.

4.1 **Operation of the facility in accordance with the proposed amendment would not involve a significant increase in the probability of occurrence or the consequences of an accident previously evaluated.**

The proposed amendment extends the period between successive refueling interval surveillance(s) to once every 24 months for those surveillance(s) evaluated herein. The proposed surveillance interval changes do not involve any change to the actual surveillance requirements, nor does it involve any change to the limits and restrictions on plant operations. The reliability of systems and components relied upon to prevent or mitigate the consequences of accidents previously evaluated is not degraded by the proposed change to the surveillance interval. Assurance of system and equipment availability is maintained. This change does not involve any change to system or equipment configuration. Therefore, this change does not increase the probability of occurrence or the consequences of an accident previously evaluated.

4.2 **Operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated.**

The proposed amendment extends the period between successive refueling interval surveillance(s) to once every 24 months for those surveillance(s) evaluated herein. The proposed surveillance interval changes do not involve any change to the actual surveillance requirements, nor does it involve any change to the limits and restrictions on plant operation. This change does not involve any change to system or equipment configuration. Therefore, this change is unrelated to the possibility of creating a new or different kind of accident from any previously evaluated.

**Oyster Creek Nuclear Generating Station (OCNGS)**  
**Operating License No. DPR-16**  
**Docket No. 50-219**  
**Technical Specification Change Request No. 203**

**4.3      Operation of the facility in accordance with the proposed amendment would not involve a significant reduction in a margin of safety.**

The proposed amendment extends the period between successive refueling interval surveillance(s) to once every 24 months (+/- 25% or 30 months) for the surveillances evaluated herein. The proposed surveillance interval changes do not involve any change to the actual surveillance requirements, nor does it involve any change to the limits and restrictions on plant operation. The reliability of systems and components is not degraded by the proposed change to the surveillance interval. Assurance of system and equipment availability is maintained. Therefore, it is concluded that operation of the facility in accordance with the proposed amendment does not involve a significant reduction in a margin of safety.

**5.0      IMPLEMENTATION:**

Implementation of this change can be made effective immediately upon receipt of the operating license amendment, as no plant modifications are necessary; however, it is requested that 60 days be allowed following receipt of the license amendment in order to make any required procedural or documentary changes.