

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	0 5 0 0 0 3 6 1	1 OF 0 1

TITLE (4)
REACTOR TRIP - MAIN STEAM ISOLATION VALVE FAILED SHUT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
0 4	1 9	8 5	8 5	0 2 8	0 0	0 5	2 0	8 5		0 5 0 0 0 0	

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)													
1	20.402(b)			20.405(c)			X 50.73(a)(2)(iv)			73.71(b)				
POWER LEVEL (10)	0 5 0			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)	
				20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
				20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)				
				20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)				
				20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)				

LICENSEE CONTACT FOR THIS LER (12)										
NAME										
J. G. HAYNES, STATION MANAGER										
TELEPHONE NUMBER										
7 1 4 4 9 2 - 7 7 0 0										

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS
F	S	B	R	G	Y					

SUPPLEMENTAL REPORT EXPECTED (14)											EXPECTED SUBMISSION DATE (15)		
YES (If yes, complete EXPECTED SUBMISSION DATE)											X NO		
											MONTH DAY YEAR		

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

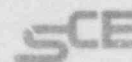
On April 19, 1985, with Unit 2 in Mode 1 at 50% power, a pressure regulator (EIIS Component Code RG) failed in the nitrogen supply to Main Steam Isolation Valve (MSIV) (EIIS Component Code ISV) 2HV-8205. The MSIV shut, isolating main steam from one steam generator (EIIS Component Code SG). The result was an imbalance in the plant operations causing an increase in loop temperature difference in the Reactor Coolant System (RCS) (EIIS Component Code AB). The Core Protection Calculators (CPCs) (EIIS Component Code CPU) generated trip signals based upon extrapolated RCS loop temperature difference of 18 degrees between cold legs. As a result, at 1224, a reactor trip was generated by the CPCs. The actual measured average cold leg temperature difference was 4.83 degrees. All essential control and safety systems functioned properly during and after the trip. The failed nitrogen pressure regulator was replaced and the MSIV tested and restored to operability.

As additional corrective action, Procedure S023-0-38, "Routine Operations and Inspections," was changed to require monitoring of MSIV nitrogen system pressure and nitrogen bottle pressure once per shift to reduce the probability of an undetected failure of the MSIV nitrogen system.

The reactor protection system functioned properly, and plant power and other important parameters were well within the envelopes of the safety analyses. Accordingly, this event had minimal safety significance. Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

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Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

May 20, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 85-028
San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Protection System. Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 85-028

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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