



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

May 31, 1985

Mr. Harold R. Denton, Director
Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

KMLNRC 85-130

Re: Docket No. STN 50-482

Ref: 1) KMLNRC 85-101 dated 05/01/85
from GIKoester, KG&E, to HRDenton, NRC
2) KMLNRC 85-112 dated 05/10/85
from GIKoester, KG&E, to HRDenton, NRC
3) KMLNRC 85-120 dated 05/17/85
from GIKoester, KG&E, to HRDenton, NRC
4) KMLNRC 85-121 dated 05/21/85
from GIKoester, KG&E, to HRDenton, NRC
5) KMLNRC 85-127 dated 05/24/85
from GIKoester, KG&E, to HRDenton, NRC

Subj: Technical Specification

Dear Mr. Denton:

Transmitted herewith are changes to the Wolf Creek Technical Specifications Table 2.2-1. Kansas Gas and Electric Company (KG&E) requests incorporation of these changes into the Wolf Creek Technical Specifications for issuance with the Wolf Creek full power license.

The changes are a result of Westinghouse's plant specific evaluation of Wolf Creek for the RdF RTD calibration concern. KG&E received the results of Westinghouse's evaluation after submittal of Reference 5.

The Wolf Creek Technical Specifications Appendix A to License No. NDF-33, (NUREG-1104) as revised by References 1, 2, 3, 4 and 5, and the above information, in my judgment, accurately reflect the plant design, FSAR and SER.

Enclosed are:

- 1) A copy of the Wolf Creek Safety Evaluation for the Increased RdF RTD Uncertainty and Rosemount Transmitter Installation prepared by Westinghouse dated May 1985 (Proprietary).

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KMLNRC 85-130

May 31, 1985

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- 2) A copy of the Wolf Creek Safety Evaluation for the Increased RdF RTD Uncertainty and Rosemount Transmitter Installation prepared by Westinghouse dated May 1985 (Non-Proprietary).

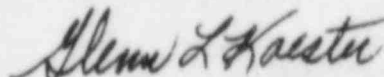
Also enclosed is a Westinghouse authorization letter CAW-85-041 and accompanying affidavit.

As this submittal contains information proprietary to Westinghouse Electric Corporation, it is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (b) (4) of Section 2.790 of the Commission's regulations.

Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10CFR Section 2.790 of the Commission's regulations. Correspondence with respect to the proprietary aspects of the Application for Withholding or the supporting Westinghouse affidavit should reference CAW-85-013 and should be addressed to R.A. Wiesemann, Manager Regulatory and Legislative Affairs, Westinghouse Electric Corporation, P.O. Box 355, Pittsburgh, Pennsylvania, 15230.

This information is hereby incorporated into the Wolf Creek Generating Station, Unit No. 1, Operating License Application.

Yours very truly,



Glenn L. Koester
Vice President, Nuclear

GLK:dab

Attach

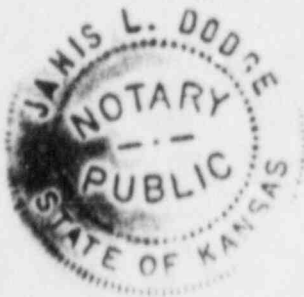
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STATE OF KANSAS)
) SS
CITY OF WICHITA)

Glenn L. Koester, of lawful age, being first duly sworn upon oath says that he is Vice President - Nuclear and an Officer of Kansas Gas and Electric Company; that he has read the foregoing document and knows the content thereof; that he has executed that same for and on behalf of said Company with full power and authority to do so; and that the facts therein stated are true and correct to the best of his knowledge, information and belief.

By Glenn L. Koester
Glenn L. Koester
Vice President - Nuclear

SUBSCRIBED and sworn to before me this 31 day of May, 1985.



Jarvis L. Dodge
Notary Public

Expiration Date 9-12-87

Attachment 1

Justifications for changes to Technical Specification Table 2.2-1

Table 2.2-1 changes are changes associated with the RdF RTD Calibration concern previously identified to the NRC by Westinghouse (reference NS-NRC-85-3034, dated 05/06/85), attached)

These changes are a result of Westinghouse's performance of an interim plant-specific evaluation for Wolf Creek. The evaluation was based on a conservative estimate of the plant specific RTD errors derived from RdF calibration data.

Based on safety evaluations performed, DNB is not expected to occur for the transients presented in the FSAR when the effects of the conservatively derived narrow range RTD errors are included. A more realistic although conservative analysis of the limiting DNB transient (Rod Withdrawal at Power) including the effects of the potential RTD errors indicates that the DNB design basis would still be met with considerable margin. The results of the safety evaluation to support plant operation confirm that the conservatively estimated RTD errors could be accommodated through the reallocation of plant specific margins although deviations from the existing technical specifications are noted. It is our judgement that there is no significant adverse impact on plant safety.