

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Catawba Nuclear Station, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 1 1 3	PAGE (3) 1 OF 0 1 3
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TITLE (4)

Retest Not Performed Following Corrective Maintenance On Isolation Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 1	1 0	8 5	8 5	0 3	8	0 0	0 7	0 3	8 5		0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)										
	20.402(b)			20.406(e)			50.73(a)(2)(iv)			73.71(b)	
	20.406(a)(1)(i)			50.73(c)(1)			50.73(a)(2)(v)			73.71(c)	
	20.406(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
	20.406(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)				
	20.406(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(vii)(B)				
20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)					
POWER LEVEL (10) 0 1 0 1 0											

LICENSEE CONTACT FOR THIS LER (12)									
NAME Roger W. Ouellette, Associate Engineer - Licensing								TELEPHONE NUMBER 7 1 0 4 3 1 7 1 3 - 1 7 1 5 1 3 0	
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)								EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)								NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 10, 1985, a work request was initiated to investigate and repair a steam leak from a Main Feedwater (CF) valve. The valve packing was adjusted on January 17, 1985, and the valve was subsequently stroked and visually inspected. However, a valve stroke time test was not performed at this time, even though this test is required under ASME Code, Section XI, subsection IWV. Technical Specification 4.0.5.a commits Catawba to adherence to this code. The valve was tested for stroke time on February 15, 1985, under routine IWV surveillance testing, leaving the valve untested for 28 days. This discrepancy was discovered on June 3, 1985.

The valve was not retested immediately following maintenance because the Planner did not indicate on the work request that a retest was required. The Planner did not follow the appropriate Maintenance Management Procedure and Station Directive when making the determination for retest. These procedures require a retest on IWV valves following maintenance. Therefore, this incident is classified as a Personnel Error. Unit 1 moved through Modes 1 through 5 while this valve was untested.

This incident is reportable pursuant to 10 CFR 50.73, Section (a)(2)(i)(B).

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Catawba Nuclear Station, Unit 1	0 5 0 0 0 4 1 3	8 5	— 0 3 8	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Main Feedwater (CF) Valve 1CF87, "Reverse Purge Isolation Valve", is a 2 inch air operated containment isolation gate valve. When opened during plant start-up, 1CF87 allows water from the Steam Generator (S/G) to flow through the Main Feedwater line in the reverse direction, bypassing the Feedwater Check Valve. This allows the feedwater line between the main feedwater nozzle and the CF isolation valve to be purged of cold water and warmed to the required 250°F before placing CF in service. 1CF87 can be manually operated from the main control board, but automatically closes on the following signals:

- Safety Injection Signal
- S/G High High Level Signal (CF Isolation)
- Reactor Trip with low Tave
- Auto start of the Auxiliary Feedwater Motor Driven Pumps
- Transfer of Plant control to the Standby Shutdown Facility
- Doghouse High-High water level

Valve 1CF87 receives an Engineered Safety Features (ESF) Actuation Signal from ESF instrumentation that is required to be operable in Modes 1 and 2. However, 1CF87 is also a containment isolation valve, which is required to be operable in Modes 1 through 4.

The testing of 1CF87 is covered under the ASME Code, Section XI, subsection IWV. Catawba is committed to adherence to this code in Technical Specification Section 4.0.5.a. The surveillance testing requirements are covered in the "Valve Inservice Testing Program", in which periodic tests are conducted on valves in accordance with the Catawba "Valve Inservice Testing Commitments".

In addition to periodic testing requirements, subsection IWV requires that, "When a valve or its control system has been replaced or repaired or has undergone maintenance that could affect its performance, and prior to the time it is returned to service, it shall be tested to demonstrate that its parameters, which could be affected by the replacement, repair, or maintenance, are within acceptable limits". This requirement is covered under Station Directive 3.2.2 and Maintenance Management Procedure 1.0 (MMP-1.0), in which the Planner is required to determine the retest requirements for the structure, system, or component following corrective maintenance, in accordance with applicable guidelines such as the "Pump and Valve Inservice Testing Program".

#### DESCRIPTION OF INCIDENT

On January 10, 1985, a Work Request was initiated to investigate and repair a steam leak on Valve 1CF87. The work was planned on January 15, and work began on January 17. To repair the valve, the packing gland was tightened under the appropriate maintenance procedure. The valve was subsequently stroked and visually inspected, but was not stroke time tested at this time. On January 18, the Job Supervisor signed the work request as completed. On February 15, 1985, a stroke time test, fail safe position test, and a remote position indication verification test were performed on 1CF87 under routine IWV surveillance testing. The testing yielded acceptable results.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
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Catawba Nuclear Station, Unit 1	0500041385	0	3	8	0	0	3 OF 03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

ICF87 was in service from January 18, 1985, to February 15, 1985, without having been stroke time tested. For the majority of the duration of inoperability, Unit 1 was in a mode which required containment isolation valves and instrumentation to be operable (Modes 1 - 4). Therefore, Technical Specification 4.0.5.a was not complied with.

When planning the Work Request, the Planner did not follow Maintenance Management Procedure 1.0 in determining retest requirements for ICF87. A retest was not performed because the Planner did not specify a retest as being required. Therefore, this incident is classified as a Personnel Error.

CORRECTIVE ACTION

- 1) A stroke time test was performed on Valve ICF87 under routine surveillance testing, yielding acceptable results.
- 2) This incident will be discussed with the responsible personnel, re-emphasizing the need to adhere to procedures; the responsible personnel will also receive retraining on Maintenance Management Procedure 1.0.

SAFETY ANALYSIS

A leak rate test is not required to be performed on valve ICF87. ICF87 is a category B valve, which means that "seat leakage in the closed position is inconsequential for fulfillment of its function". The feedwater lines are connected to the secondary side of the steam generator, which is kept at a higher pressure than the primary side soon after a postulated LOCA occurs. Any leakage between the primary and secondary sides of the steam generator is directed inward to containment. Also, tempering flow isolation valve ICA188, serves as redundant isolation for the reverse purge line in the case of failure of valve ICF87.

The subsequent stroke time test on ICF87 yielded acceptable results, but even in the event of failure of ICF87, containment integrity would not have been adversely affected. The health and safety of the public were not affected by this incident.

**DUKE POWER COMPANY**

P.O. BOX 33189

CHARLOTTE, N.C. 28242

HAL B. TUCKER

VICE PRESIDENT  
NUCLEAR PRODUCTION

TELEPHONE  
(704) 373-4531

July 3, 1985

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Subject: Catawba Nuclear Station, Unit 1  
Docket No. 50-413

Gentlemen:

Pursuant to 10 CFR 50.73 Section (a) (1) and (d), attached is Licensee Event Report 413/85-38 concerning a retest not being performed following corrective maintenance on a Main Feedwater isolation valve. This event was considered to be of no significance with respect to the health and safety of the public.

Very truly yours,

*H. B. Tucker*

Hal B. Tucker

RWO:slb

Attachment

cc: Dr. J. Nelson Grace, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, NW, Suite 2900  
Atlanta, Georgia 30323

Palmetto Alliance  
2135½ Devine Street  
Columbia, South Carolina 29205

Mr. Jesse L. Riley  
Carolina Environmental Study Group  
854 Henley Place  
Charlotte, North Carolina 28207

Robert Guild, Esq.  
P. O. Box 12097  
Charleston, South Carolina 29412

American Nuclear Insurers  
c/o Dottie Sherman, ANI Library  
The Exchange, Suite 245  
270 Farmington Avenue  
Farmington, CT 06032

M&M Nuclear Consultants  
1221 Avenue of the Americas  
New York, New York 10020

INPO Records Center  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Resident Inspector  
Catawba Nuclear Station

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