

EXHIBIT 2B

1985 Information Collection Budget - Supporting Exhibit: New Information Collections Proposed in 1985 Nuclear Regulatory Commission

<u>ICB Form No.</u>	<u>Title</u>	<u>9/85 Estimate (Hours)</u>
1	Interim IDVP (Independent Design Verification Program), 10 CFR 50, Appendix B	36,000

The NRC staff will request an applicant for an operating license to provide additional assurance that the design process used in constructing the plant has fully complied with NRC regulations and licensing commitments.

2	Interim Requirements for Nonpower Reactors Possessing Category I Quantities of Materials, 10 CFR 73	30
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This rulemaking is being proposed by staff in response to the Commission's direction to consider if any additional requirements should be placed on Category I nonpower reactors. Staff is proposing to require notification and a description of the additional measures to be implemented at least 48 hours before a licensee is no longer exempt from the Category I physical protection requirements.

3	Improvement of Category II Physical Protection Requirements for HEU, 10 CFR 73	2,700
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In response to the Commission's direction to expeditiously improve security at nonpower reactors possessing Category II quantities of high enriched uranium, rulemaking is being proposed by staff. The amended regulations would require the licensee to submit a revised security plan describing the additional security measures to be implemented. Beyond this one-time submittal, no additional reporting is anticipated.

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4

Proposed Revisions to Notification Requirements for
Safeguards Events, 10 CFR 73.71

8,300

The proposed rule would clarify the safeguards event reporting requirements for NRC licensees and would improve the NRC safeguards events data base by requiring more uniform safeguards event reports.

5

Insider Safeguards Rules Package,
10 CFR 50 and 73

81,990

Three related proposed rulemaking actions to: (1) establish personnel screening requirements for power reactor licensee employees and contractors, (2) clarify the role of pat-down entry searches, and (3) revise vital area designation and protection, access control to vital areas and key and lock control.

6

Notification of Transient, Export and Import Shipments of
Natural Uranium, 10 CFR 40

45

Pursuant to the Convention on the Physical Protection of Nuclear Material, licensees will be required to provide advance notification and, in some cases, assurances of protection for transient, export and import shipments of natural uranium in amounts exceeding 500 kilograms.

7

Notification of Transient Shipments of Nuclear Material,
10 CFR 70

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Pursuant to the Convention on the Physical Protection of Nuclear Material, licensees will be required to provide advance notification and, in some cases, assurances of protection for transient shipments of formula quantities of strategic special nuclear material, special nuclear material of moderate and low strategic significance, and irradiated reactor fuel.

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- 8 Notification of Export and Import of Shipments of Nuclear Material, 10 CFR 73

925

Pursuant to the Convention of the Physical Protection of Nuclear Material, licensees will be required to provide advance notification and, in some cases, assurances of special nuclear material, special nuclear material of moderate and low strategic significance, and irradiated reactor fuel.

- 9 Survey of Licensees Performing Leak Testing of Radioactive Sources Containing Byproduct Material, 10 CFR 30

20

In connection with a special study, NRC plans to survey selected licensees to determine whether the regulatory requirement for leak test frequency of six months for byproduct materials sources should be continued, or whether the leak test frequency requirements should be extended to annually for certain categories of licensees.

- 10 Annual Report of Individual Radiation Exposure (NRC Form 10)
10 CFR 20

4,275

The new form would provide the NRC with an annual summary of radiation exposure records to provide better information on the doses being received in licensed programs including information on the doses received by transient and moonlighting workers. The annual report would replace the annual statistical reports presently required by 10 CFR 20.407 and the termination reports presently required by 10 CFR 20.408.

- 11 Emergency Preparedness Procedures for Fuel Cycle and Other Radioactive Materials Licensees, 10 CFR 30, 40, 70, and 72

30,000

The proposed rule would require specified fuel cycle facility and byproduct materials licensees to submit emergency preparedness procedures. Local governments with jurisdiction over these facilities would also have to submit emergency response plans to FEMA.

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12

Decommissioning Criteria for Nuclear Facilities,
10 CFR 30, 40, 50, 51, 70, and 72

31,600

The proposed rule would require NRC licensees to submit a plan providing for the financial assurance necessary for decommissioning nuclear facilities as well as providing for the maintenance of records which could affect decommissioning.

13

Licenses and Radiation Safety Requirements for Well-Logging
Operations, 10 CFR 39

850

The proposed rule would provide application, reporting and recordkeeping requirements for persons who desire to obtain an NRC license to perform operations such as well-logging, material-logging, radioactive marking, and subsurface use of radioactive materials in tracer studies.

14

Acceptance Criteria for Emergency Core Cooling Systems for
Light-Water-Cooled Nuclear Power Plants, 10 CFR 50

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The Commission is considering changing certain technical and nontechnical requirements within the existing ECCS rule (10 CFR 50.46). The nontechnical changes would be procedure-oriented and would, among other things, allow for corrections to be made to vendor ECCS analyses codes during the construction permit review and during construction of nuclear power plants.

15

Fitness for Duty of Personnel with Access to Nuclear Power
Plants, 10 CFR 50.54

1,400

The rule would require holders of operating licenses for nuclear power plants to establish and implement controls to provide reasonable assurance that personnel with access to vital areas of the nuclear power plants are fit for duty.

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| 16 | Codes and Standards for Nuclear Power Plants (Through Summer 1984),
10 CFR 50.55a | 1,500 |
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The proposed rule would apply to existing and future holders of construction permits for nuclear power plants. The proposed rule would require additional testing procedures for nuclear power plant materials and equipment. Records must be kept of these test.

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| 17 | High-Enriched Uranium (HEU) Requirements for Domestic Non-Power
Reactors, 10 CFR 50.64 | 3,720 |
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The proposed rule would require that non-power reactors either convert to low-enriched uranium fuel on an approved schedule or apply for a determination that the non-power reactor has a unique purpose requiring continued use of HEU.

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| 18 | Criteria and Procedures for Determining the Adequacy of
Available Spent Nuclear Fuel Storage Capacity, 10 CFR 53 | 2,000 |
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Any person owning and operating a civilian nuclear power reactor, when requesting a determination under Sec. 135(b) of the Nuclear Waste Policy Act of 1982, must submit general and specific information to assist the NRC in determining whether that person cannot reasonably provide adequate spent nuclear fuel storage capacity.

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| 19 | Disposal of High-Level Radioactive Wastes in Geologic
Repositories: Procedural Amendments, 10 CFR 60 | 460 |
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The Nuclear Waste Policy Act of 1982 and 10 CFR Part 60 contain detailed provisions for the participation of States and Indian tribes, upon written request, in the process of siting and developing a high-level radioactive waste geologic repository.

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- 20 Assessment of Containment Surrogates for Severe Accident
Decision-Making

6,400

Severe accident regulatory decisions may address plants as individual entities or as broad classes depending on how well individual plants may be represented by surrogates. In order to make these decisions detailed information about containment designs will be needed. This information is beyond the detail submitted in an SAR but should be available to licensees and applicants.

- 21 Implementation of the Resolution of USI A-40 Seismic
Design Criteria, A Short-Term Program, 10 CFR 50

16,000

The proposed resolution of USI-40 requires all applicants to re-evaluate the seismic design of Class I free standing tanks. The result of this re-evaluation are to be submitted to the NRC for review and determination as to whether seismic design criteria have been met.

22. Information Necessary for Development of the Resolution of
USI A-47 Safety Implications of Control Systems

1,000

Most of the plant-specific information required to develop the resolution for the USI has been obtained. Some additional plant-specific data for a few plants in the form of a request to confirm data used in the various analyses may be necessary.

- 23 Operator Survey

200

The purpose of this survey is to determine concerns and effects of regulations and proposed requirements on licensed reactor operators.

The NRC will use the information in the development and review of regulations which most directly affect the operations crew at nuclear power plants; e.g., overtime, shift work issues and educational requirements.

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24. Integrated Implementation Schedule (priorities for new and existing requirements for power reactor licensees)

11,160

Based on the NRC's Statement of Policy and Planning Guidance for 1984, management of safety-related modifications, both utility-initiated and NRC-required, at operating reactors, including the optimum allocation of resources, has become a major consideration in licensing activities. This effort is the first step toward development of an industry-wide approach to achieve effective management of plant changes and optimum use of resources. The NRC has determined that it is necessary for each utility to make its intentions known with respect to the development of integrated implementation schedules.

25. Proposed Rulemaking on Training, Qualifications and Examinations for Nuclear Power Plant Personnel, 10 CFR 50 and 55

28,900

These proposed new information collection requirements result from Section 306 of the NWSA of 1982, P.L. 97-425, directing the NRC to: promulgate regulations or guidance for the training and qualifications of civilian nuclear power plant operators, supervisors, technicians and other operating personnel; establish simulator training requirements for applicants for operator's licenses and for operator requalification programs; and establish requirements for operating tests at civilian nuclear power plant simulators.

26. Implementation of Steam Generator Generic Requirements (Tube Degradation and Tube Rupture Events)

28,100

The staff has developed additional requirements related to steam generator integrity and mitigation of the consequences of steam generator tube rupture (SGTR) events. The requirements are based on the issues related to the resolution of U.S.I.'s A-3, A-4, and A-5 regarding steam generator tube integrity and the January 25, 1982 SGTR event at the R.E. Ginna plant. The recommendations are also the result of plant specific operating experience as discussed in NUREG-0886 and in various plant specific licensing actions.

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27.	Implementation of the Resolution of USI A-43 "Containment Emergency Sump"	5,000
The proposed resolution of USI A-43 requires all licensees of operating plants to re-evaluate the containment sump design in order to reconfirm containment emergency sump operability in the post LOCA period in accordance with the proposed Standard Review Plan Section 6.2.2., Revision 4 and Regulatory Guide 1.82, Revision 1.		
28.	Information Necessary for Development of the Resolution of A-45 "Shut Down Decay Heat Removal Requirements"	2,000
The development of the proposed resolution for A-45 will require extensive knowledge about decay heat removal systems. Although many plant systems are generally similar, there are significant differences regarding potential vulnerability to severe external hazards. Much of the information has been obtained through existing documentation. A formal request to the licensees may be necessary to obtain plant-specific information to complete the resolution.		
29.	Implementation of the Resolution of USI-44 "Station Blackout"	150,000
The proposed resolution of USI-44 will require all licensees to address the proposed new rule and the guidelines in a new regulatory guide to demonstrate the ability to cope with a total loss of AC power for a specified time.		
30.	Implementation of the Resolution of USI-46 "Seismic Qualification of Equipment"	12,000
The proposed resolution of USI-46 will require licensees of operating plants to provide documentation on seismic qualification of electrical and mechanical equipment.		

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| 31. | Implementation of the Proposed Rule Addressing USI-49
"Pressurized Thermal Shock" | 20,000 |
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Licensees will provide information on fracture toughness of pressure vessel materials in FY 1985 in accordance with the requirements of the proposed rule.

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| 32. | Procedures Generation Packages
Required by TMI Action Plan | 72,800 |
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Licensees and applicants will submit Procedures Generation Packages (PGP). These PGPs are the mechanisms for documenting and explaining to the NRC staff the licensees' program for developing upgraded Emergency Operating Procedures (EOPs), the basis for the upgraded EOPs, the methods and objectives of the program for assuring the validity of the upgraded EOPs, and operator training on the upgraded EOPs.

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| 33. | Installation of Post Accident Monitoring Instrumentation,
Regulatory Guide 1.97 | 119,000 |
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Licensees will be required to submit a report to NRC describing how they conform to the guidance of Regulatory Guide 1.97, Revision 2. This report should include the justification for any exceptions or deviations from the guide. This action is required via GL 82-33 (NUREG-0737 Supplement 1).

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| 34. | BWR Water Level Instrumentation Design Changes | 8,640 |
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The NRC and BWR Owners have identified a need to improve the water level instrumentation system. The BWR Owners contracted Saul Levy, Inc., to determine the best method of improving the instrumentation system. Saul Levy proposed several design changes to accomplish the goals. The requirement is to determine which is best for each plant and submit the plant-specific design to NRC along with a schedule for implementation.

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35.	BWR Pipe Cracks - NUREG-0313	2,640
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BWR licensees will be requested to provide information that will be used to determine the licensees' compliance with the guidelines set forth in NUREG-0313, Revision 2. Request for additional information may be required if the licensees' responses are considered inadequate.

36.	Reactor Operator and Senior Reactor Operator License Training Programs	1,220
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Training information on the reactor operator training program is required to enable the NRC staff to develop operator licenses and requalification audit examinations. Submittal of this information is mandatory. Information consists of lesson plans, system descriptions, and other training material used in training and requalification programs.

37.	Questionnaire on Radiation Survey Instrument Calibration	13,774
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The questionnaire would gather information from NRC licensees on the instruments used to survey radiation levels at licensee sites to determine the adequacy of the instruments and to determine if upgraded criteria are required.

TOTALS	704,679
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