

October 16, 1996

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Mr. Lew W. Myers
Vice President Nuclear - Perry
Centerior Service Company
P. O. Box 97, A200
Perry, OH 44081

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - DRYWELL BYPASS LEAKAGE TEST
EXTENSION, PERRY NUCLEAR POWER PLANT, UNIT NO. 1 (TAC NO. M94493)

Dear Mr. Myers:

By letter dated January 16, 1996, you submitted a proposed license amendment to extend the test interval for the drywell bypass leakage test from 18 months to 10 years for the Perry Nuclear Power Plant, Unit No. 1. The NRC staff has performed an initial review of your submittal, and needs additional information to complete its review. Please respond to the enclosed request for additional information (RAI) within 45 days of receiving this letter.

Sincerely,

Original signed by:

Jon B. Hopkins, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-440

Enclosure: RAI

cc w/encl: See next page

DOCUMENT NAME: G:\PERRY\DRYWELL.RAI

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L. Myers
Centerior Service Company

cc:

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Perry Nuclear Power Plant
Unit Nos. 1 and 2

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The Honorable Lawrence Logan
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Perry, Ohio 44081

The Honorable Robert V. Orosz
Mayor, Village of North Perry
North Perry Village Hall
4778 Lockwood Road
North Perry Village, Ohio 44081

Attorney General
Department of Attorney General
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Radiological Health Program
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Ohio Environmental Protection
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Mr. Thomas Haas, Chairman
Perry Township Board of Trustees
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Mr. Richard D. Brandt, Plant Manager
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REQUEST FOR ADDITIONAL INFORMATION

PERRY NUCLEAR POWER PLANT

DRYWELL LEAK RATE TESTING

1. Page 7 of Attachment 1 to the January 16, 1996, submittal states that leakage was identified past "boot seals" that are installed in the annular space between the safety relief valve discharge lines and the drywell wall. The submittal states that short-term action was to replace leaking seals. The submittal also stated that longer term action were being considered. Please discuss the status of this issue. Have the leaking seals been replaced? What longer term actions have been taken or are planned?
2. Table 2 shows that two tests were performed in 1994 with leakage rates of more than an order of magnitude difference. Discuss why two tests were run one month apart and the reasons for the difference between the leakage rate test results.
3. Provide a list of all drywell isolation valves. Provide diameters of the valves. Indicate which valves close automatically on indication of a LOCA and which valves are locked closed when drywell integrity is required. Indicate which valves have position indications in the control room. For those valves which have neither automatic isolation or are not locked closed, what assurance is there that the valves will be shut if a LOCA occurs? Can it be demonstrated that all valves below a certain diameter can be open during a LOCA without exceeding the design basis A/ \sqrt{K} ? If relevant material is in the UFSAR, a reference is sufficient.
4. Provide a commitment to perform a qualitative assessment at least once per operating cycle and specify the method to be used. The licensee should determine and the commitment should state that the method will provide reasonable assurance of the ability of the drywell to perform its design basis pressure suppression function, that is, that the drywell is operable.
5. (a) Describe the procedures that are used to assure that, following a refueling outage or any other outage during which the drywell may be open, that all penetrations are properly isolated, and that no damage has been done, for example, to seals or valves which could cause excessive leakage. (b) Specifically discuss the drywell equipment hatch. What assurance is there, if no drywell bypass leakage rate test is performed, that the drywell equipment hatch is not leaking excessively?
6. Demonstrate, if possible, that one purge valve can be left open without exceeding the containment failure pressure.
7. What requirements are there for leakage rate testing of the drywell after modifications to the drywell structure or penetrations?

Enclosure