

May 13, 1985

Docket Nos. 50-282
and 50-306

Mr. D. M. Musolf
Nuclear Support Services Department
Northern States Power Company
414 Nicollet Mall
Midland Square - 4th Floor
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

SUBJECT: SAFETY EVALUATION AND REQUEST FOR ADDITIONAL INFORMATION
GENERIC LETTER 82-28; NUREG-0737 ITEM II.F.2, INADEQUATE
CORE COOLING INSTRUMENTATION

The Commission has completed the review of your letters dated March 14, 1983, and April 2, July 6 and September 21, 1984 responding to Generic letter (GL) 82-28, Inadequate Core Cooling Instrumentation (ICCI) systems, for the Prairie Island Nuclear Generating Plant Unit Nos. 1 and 2. Our evaluation shows that the subcooling Margin Monitor and the core exit thermocouples (CET) upon completion of the upgrade of the backup CET display are acceptable. However, we request that you provide additional information regarding the reactor vessel level instrument system (RVLIS) as delineated in Enclosure 1 of our Safety Evaluation.

In order to complete our review of your Inadequate Core Cooling instrument system, we have scheduled a site audit on June 20, 1985 to review the design, functional test results (Unit 1 only), and operating procedures. The details of this site visit are being coordinated with your site personnel through the resident inspector.

We conclude that additional information described in Enclosure 1 of our Safety Evaluation is necessary in order to complete our evaluation. Therefore, we request that you provide a response to each item within 45 days from your receipt of this letter. Your response is to include a model technical specification change specific for Prairie Island covering the requirements of the sub-cooling margin, the core exit thermocouples, and the RVLIS.

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The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

A copy of our Safety Evaluation is enclosed.

Sincerely,

Original signed by:

James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosure:
As stated

cc w/enclosure
See next page

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