

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
EDWIN I. HATCH, UNIT II

DOCKET NUMBER (2)

0 5 0 0 0 3 6 6 1 OF 0 2

PAGE (3)

TITLE (4)

## MAIN STEAM SRV OUT OF TOLERANCE DURING TESTING

| EVENT DATE (5) |     |      | LER NUMBER (6) |                   |                 | REPORT DATE (7) |     |      | OTHER FACILITIES INVOLVED (8) |                  |   |   |   |   |   |   |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|------------------|---|---|---|---|---|---|
| MONTH          | DAY | YEAR | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH           | DAY | YEAR | FACILITY NAMES                | DOCKET NUMBER(S) |   |   |   |   |   |   |
| 0              | 4   | 2    | 5              | 8                 | 5               | 8               | 5   | 0    | 1                             | 6                | 0 | 0 | 0 | 0 | 0 | 0 |
| 0              | 4   | 2    | 5              | 8                 | 5               | 8               | 5   | 0    | 1                             | 6                | 0 | 0 | 0 | 0 | 0 | 0 |

| OPERATING MODE (9) |       | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11) |   |                  |  |                      |  |  |  |  |  |  |  |
|--------------------|-------|--|---|------------------|--|----------------------|--|--|--|--|--|--|--|
| POWER LEVEL (10)   | 0 0 0 | 20.402(b)  |   | 20.406(c)        |  | 50.73(a)(2)(iv)      |  | 73.71(b)   |  |  |  |  |  |
|                    |       | 20.406(a)(1)(i)  |   | 50.36(c)(1)      |  | 50.73(a)(2)(v)       |  | 73.71(c)   |  |  |  |  |  |
|                    |       | 20.406(a)(1)(ii)   |   | 50.36(c)(2)      |  | 50.73(a)(2)(vii)     |  | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |  |  |  |  |  |
|                    |       | 20.406(a)(1)(iii)  | X | 50.73(a)(2)(i)   |  | 50.73(a)(2)(viii)(A) |  |  |  |  |  |  |  |
|                    |       | 20.406(a)(1)(iv)   |   | 50.73(a)(2)(ii)  |  | 50.73(a)(2)(viii)(B) |  |  |  |  |  |  |  |
|                    |       | 20.406(a)(1)(v)  |   | 50.73(a)(2)(iii) |  | 50.73(a)(2)(ix)      |  |  |  |  |  |  |  |

LICENSEE CONTACT FOR THIS LER (12)

| NAME   | TELEPHONE NUMBER      |
|--|-----------------------|
| Steven B. Tipps, Superintendent of Regulatory Compliance | 9 1 2 3 6 7 1 7 8 5 1 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |   |  |  |  |  |  |  |  |  |  |
|-------|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|---|--|--|--|--|--|--|--|--|--|
| X     | S      | B         | I            | R                 | V     | T      | O         | 2            | 0                 | Y |  |  |  |  |  |  |  |  |  |

SUPPLEMENTAL REPORT EXPECTED (14)

| YES (If yes, complete EXPECTED SUBMISSION DATE) | NO                       | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|--------------------------|-------------------------------|-------|-----|------|
| <input checked="" type="checkbox"/>             | <input type="checkbox"/> |                               |       |     |      |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

With the unit in cold shutdown for refueling, and during performance of Wyle Laboratories' testing of main steam safety relief valves (SRVs), four SRVs failed to lift within the  $\pm 1\%$  tolerance range required by Tech. Specs. section 3.4.2.1.

The Target Rock model 7567F SRVs used on Unit 1 and Unit 2 are the most accurate and reliable SRVs available. However, they are certified only to a  $\pm 3\%$  tolerance by Target Rock. The cause of these valves' failure to meet the  $\pm 1\%$  tolerance allowed by Tech. Specs. is valve pilot setpoint drift.

The SRV pilot assemblies were subsequently refurbished, and the SRVs were satisfactorily retested. The retesting results demonstrated that the SRVs met the  $\pm 1\%$  tolerance range requirement of Tech. Specs. section 3.4.2.1.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

|                         |                   |                |                   |     |                 |   |    |          |  |  |  |
|-------------------------|-------------------|----------------|-------------------|-----|-----------------|---|----|----------|--|--|--|
| FACILITY NAME (1)       | DOCKET NUMBER (2) | LER NUMBER (8) |                   |     |                 |   |    | PAGE (3) |  |  |  |
|                         |                   | YEAR           | SEQUENTIAL NUMBER |     | REVISION NUMBER |   |    |          |  |  |  |
|                         |                   |                |                   |     |                 |   |    |          |  |  |  |
| EDWIN I. HATCH, UNIT II | 0 5 0 0 0 3 6 6   | 8 5            | — 0 1 6           | — 0 | 0 0             | 2 | OF | 2        |  |  |  |

TEXT (If more space is required, use additional NRC Form 365A's) (17)

This 30 day LER is required by 10CFR 50.73(a)(2)(i)(B) because the requirements of Tech. Specs. section 3.4.2.1 could not be met.

The unit was in cold shutdown for a refueling outage, when the main steam safety relief valves (SRVs) were removed and shipped to Wyle Laboratories for testing. During testing of the Unit's SRVs on 04/25/84, it was determined that four SRVs (listed below) failed to lift within the  $\pm 1\%$  tolerance required by Tech. Specs. section 3.4.2.1.

| SRV's Listed by<br>Master Parts<br>List Numbers | Expected Lift<br>Setpoint in<br>PSIG | As found Lift<br>Pressure In PSIG<br>First Test | % Out of Tech.<br>Specs. Tolerance | As Left<br>Lift Pressure<br>In PSIG After<br>Repair |
|---|--------------------------------------|---|------------------------------------|---|
| 2B21-F013C                                      | 1090                                 | 1070  | 0.83                               | 1093  |
| 2B21-F013D                                      | 1100                                 | 1066  | 2.09                               | 1106  |
| 2B21-F013G                                      | 1090                                 | 1107  | 0.56                               | 1092  |
| 2B21-F013H                                      | 1100                                 | 1132  | 1.91                               | 1103  |

The Target Rock model 7567F safety relief valves used on Unit 1 and Unit 2 are the most accurate and reliable SRVs available. However, they are certified only to a  $\pm 3\%$  tolerance by Target Rock. The cause of these valve's failure to meet the  $\pm 1\%$  tolerance allowed by Tech. Specs. is valve pilot setpoint drift. Consequently, past corrective action has not prevented recurrence.

A request for a revision of Unit 1, and Unit 2 Tech. Specs. has been submitted. Approval of the Tech. Specs. revision will change the acceptance criteria from  $\pm 1\%$  tolerance to a  $\pm 3\%$  tolerance, and is expected to prevent recurrence of this type event.

The four SRVs which failed to meet the  $\pm 1\%$  Tech. Specs. acceptance criteria had their respective pilot assemblies refurbished and retested. SRV retesting after refurbishing verified that the SRV's would meet the requirement of Tech. Specs. section 3.4.2.1. (refer to "AS LEFT" column above). All SRV's will be installed prior to Unit startup.

Other past similar events were reported via LER 50-321/1984-024 and 50-366/1984-002.

This event had no impact upon any other system in Unit 2, or Unit 1.

The health and safety of the public were not affected by this event.

Georgia Power Company  
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Baxley, Georgia 31513  
Telephone 912 367-7781  
912 537-9444



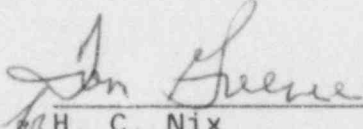
Edwin I. Hatch Nuclear Plant

May 21, 1985  
GM-85-478

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-016. This report is required by 10CFR 50.73(a)(2)(i).

  
H. C. Nix  
General Manager

*SB1*  
HCN/SBT/vlz

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