



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket  
File

February 18, 1993

Docket No. 50-245

Mr. John F. Opeka  
Executive Vice President, Nuclear  
Connecticut Yankee Atomic Power Company  
Northeast Nuclear Energy Company  
Post Office Box 270  
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT 1 - INSERVICE TESTING  
PROGRAM RELIEF REQUESTS FOR PUMPS AND VALVES (TAC NO. M84917)

By letter dated October 30, 1992, Northeast Nuclear Energy Company (NNECO) provided the NRC staff with their resolution of the outstanding items regarding the Inservice Testing (IST) Program at Millstone Unit No. 1. By letter dated January 8, 1992, the staff issued its safety evaluation in response to NNECO's submittal of Revision 4 of the Millstone Unit No. 1 IST Program and associated relief requests. In the NRC's January 8, 1992, safety evaluation, the staff requested NNECO to provide a description of the process used in developing the IST Program. In addition, the staff requested that NNECO address the items identified in the "Anomalies and Action Items" Section of the above-mentioned safety evaluation. NNECO's October 30, 1992, submittal addressed these items and, in addition, forwarded Revision 5 of the IST Program (with 8 relief requests for pumps and 14 relief requests for valves). The staff's review of NNECO's submittal is provided in the enclosed safety evaluation.

The Code of Federal Regulations, 10 CFR 50.55a, requires that inservice testing of certain ASME Code Class 1, 2, and 3 pumps and valves be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda, except where relief has been granted or proposed alternatives authorized by the Commission pursuant to 50.55a(f)(6)(i), (a)(3)(i), or (a)(3)(ii). Guidance on acceptable alternatives to Section XI requirements was provided in Generic Letter 89-04, Guidance on Developing Acceptable Inservice Testing Programs, for certain aspects of IST.

The staff concludes that some of the relief requests as evaluated and modified in the enclosed safety evaluation will provide reasonable assurance of the operational readiness of the pumps and valves to perform their safety-related functions, provided NNECO makes the applicable modifications described in the relief requests during refueling outages 14 (1994) and 15 (1996). The staff has determined that granting relief pursuant to 10 CFR 50.55a(f)(6)(i) is

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Mr. John F. Opeka

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authorized by law and will not endanger life or property, or the common defense and security and is otherwise in the public interest, giving due consideration to the burden upon the licensee that could result if the requirements were imposed on the facility.

For certain other items, relief is not granted and for those items the staff has requested the licensee to perform additional review and to address the related issues within 6 months from the date of the enclosed safety evaluation. These items are as follows: Sections 2.2, 2.3, 2.6, 3.9, and 3.12, of the enclosed safety evaluation. Other sections may include actions NNECO is to take to address provisions in the granting of relief. These provisions should be incorporated into the inservice testing program within 6 months from the date of the enclosed safety evaluation unless otherwise noted in the safety evaluation.

This request for information affects one respondent, and, therefore, is not subject to Office of Management and Budget review under P.L. 96-511.

Sincerely,

Original signed by

John F. Stolz, Director  
Project Directorate 1-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
As stated

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