1.0 INTRODUCTION AND GENERAL DESCRIPTION OF PLANT

1.1 INTRODUCTION

The Westinghouse Electric Corporation (hereinafter referred to as Westinghouse) has developed this Reference Safety Analysis Report (RESAR-SP/90) for the Westinghouse Advanced Pressurized Water Reactor (WAPWR) as part of its continuing efforts toward design and licensing standardization of nuclear power plants. RESAR-SP/90 is a standard safety analysis report submitted initially for Preliminary Design Approval (PDA) in accordance with Appendix O, "Standardization of Design; Staff Review of Standard Designs," to Part 50 of Title 10 of the Code of Federal Regulations (hereinafter referred to as 10CFR). The ultimate objective is to obtain a Final Design Approval (FDA) of RESAR-SP/90 followed by a rulemaking proceeding and design certification.

1.2 GENERAL PLANT DESCRIPTION

1.2.2 Principal Design Criteria

RESAR-SP/90 is designed to comply with 10 CFR Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants." The specific applications of General Design Criteria to RESAR-SP/90 are discussed in Section 3.1 of RESAR-SP/90 Module 7, "Structural/Equipment Design.

1.6 MATERIAL INCORPORATED BY REFERENCE

The WAPWR Radiation Protection Module incorporates, by reference, certain topical reports. The topical reports, listed in Table 1.6-1, have been filed previously in support of other Westinghouse applications.

The legend for the review status code letter follows:

A - U.S. Nuclear Regulatory Commission review complete; USNRC acceptance letter issued.

AE - U.S. Nuclear Regulatory Commission accepted as part of the Westinghouse emergency core cooling system (ECCS) evaluation model only; does not constitute acceptance for any purpose other than for ECCS analyses.

B - Submitted to USNRC as background information; no undergoing formal USNRC review.

O - On file with USNRC: older generation report with current validity; not actively under formal USNRC review.

U - Actively under formal USNRC review.

TABLE 1.6-1
MATERIAL INCORPORATED BY REFERENCE

Westinghouse Topical Report No.	Title	Revision Number	SAR Section Reference	Submitted to the NRC	Review Status
WCAP-8370	Westinghouse Water Reactor Divisions Quality Assurance Plan	Rev. 9A Amend. 1	17.1	2/81	A

1.8 CONFORMANCE WITH THE STANDARD REVIEW PLAN

In accordance with 10CFR50.34(g), Table 1.8-1 of each PDA module identifies and evaluates deviations from the acceptance criteria of those sections of the NRC Standard Review Plan (NUREG-0800) pertinent to the subject module. Table 1.8-1 provides this list for the "Radiation Protection".

TABLE 1.8-1

STANDARD REVIEW PLAN DEVIATIONS

SRP Acceptance Criteria	Deviation	Section
SRP 12.3 12.4 Describe the radiation instrumentation that will be used to meet the criticality accident monitoring requirements of § 70.24 of 10 CFR Part 70 for storage area for new fuel.	The design of the fuel pool racks precludes criticality under all postulated normal and accident conditions. Therefore, criticality monitors, as required by 10 CFR 70.24 and Regulatory Guide 8.12, are not needed.	12.3.4.1