

## LICENSEE EVENT REPORT (LER)

APPROVED ONS NO. 3190-0104  
EXPIRES - 8/31/85

FACILITY NAME (1)

Limerick Generating Station - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 5 2

PAGE (3)

1 OF 0 3

TITLE (4)

Automatic Isolation of the Reactor Water Cleanup System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 5	0 7	8 5	8 5	0 5 1	0 0 0	5	2	9 8 5			0 5 0 0 0 1 1
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)								
4			<input checked="" type="checkbox"/> 00.400(a) <input type="checkbox"/> 00.400(b) <input type="checkbox"/> 00.400(c) <input type="checkbox"/> 00.400(d) <input type="checkbox"/> 00.400(e) <input type="checkbox"/> 00.400(f) <input type="checkbox"/> 00.400(g) <input type="checkbox"/> 00.400(h) <input type="checkbox"/> 00.400(i) <input type="checkbox"/> 00.400(j) <input type="checkbox"/> 00.400(k) <input type="checkbox"/> 00.400(l) <input type="checkbox"/> 00.400(m) <input type="checkbox"/> 00.400(n) <input type="checkbox"/> 00.400(o) <input type="checkbox"/> 00.400(p) <input type="checkbox"/> 00.400(q) <input type="checkbox"/> 00.400(r) <input type="checkbox"/> 00.400(s) <input type="checkbox"/> 00.400(t) <input type="checkbox"/> 00.400(u) <input type="checkbox"/> 00.400(v) <input type="checkbox"/> 00.400(w) <input type="checkbox"/> 00.400(x) <input type="checkbox"/> 00.400(y) <input type="checkbox"/> 00.400(z)								
POWER LEVEL (10)			73.71(b) 73.71(c) OTHER (Specify in Abstract below and in Title, NRC Form 200A)								
0 0 0											

LICENSEE CONTACT FOR THIS LER (12)

NAME

John C. Nagle, Senior Engineer - Special Projects

TELEPHONE NUMBER

AREA CODE

2 1 5 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If you complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1000 words, i.e., approximately fifteen single-spaced typewritten lines) (16)

Abstract: 85-051

On May 7, 1985, at 1012 hours with Unit 1 in cold shutdown, the reactor water cleanup (RWC) system inboard suction valve closed to the isolation position when a simulated high differential flow signal was inserted into the isolation logic during surveillance testing. The cause of the event was poor communication between the instrument and control technicians and control room operators during the functional test of the flow differential switch of the RWC system inboard suction valve. The isolation signal was cleared and the RWC system was returned to service at 1025 hours the same day.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO 3190-0104

EXPIRES 8/31/85

FACILITY NAME (1)

Limerick Generating Station  
Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 5 2

LER NUMBER (3)

YEAR

SEQUENTIAL  
NUMBERREVISION  
NUMBER

PAGE (3)

85

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0 2 OF

0 3

TEXT (If more space is required, use additional NRC Form 366a (1))

Description of the Event:

On May 7, 1985, at 1012 hours with Unit 1 in cold shutdown, during surveillance testing of Flow Differential Switch FDS-44-1N603A, the reactor water cleanup system inboard suction valve, HV-44-1F001, received a high differential flow isolation signal. Valve HV-44-1F001 moved to the closed position. After the high differential flow isolation was cleared, the reactor water cleanup system was returned to normal operation at 1025 hours. The EIIIS Code designation for this system is CE.

Consequences of the Event:

The RWCU system isolated properly. There were no adverse effects. Reactor water chemistry was not affected due to the short duration of the isolation.

Cause of the Event:

The event was caused by poor communication between the instrument and control (I&C) technicians and the control room operators during performance of surveillance test ST-2-044-600-1, "NSSSS-RWCU DIFF FLOW-HIGH; FUNCTIONAL TEST (FDS-44-1N603A)". During set-up of the surveillance test, the I&C technicians informed the reactor operator (RO) and chief operator (CO) that the power supply feeder breaker to the RWCU system inboard suction valve HV-44-1F001 would have to be opened during the performance of the test. The RO and CO misunderstood this information and immediately opened the breaker. Approximately 20 minutes later, the I&C technicians notified the CO that the technicians were ready to start the test. The CO mistakenly understood that the test was complete and assumed that the breaker could be restored immediately to normal. The CO observed that the "RWCU High Differential Flow Isolation Timer Initiated" annunciator was clear and directed that the breaker be closed. Shortly after this action, a simulated high differential flow signal was inserted (as part of the functional test) and the RWCU system inboard suction valve closed.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8-31-85

FACILITY NAME (1)

Limerick Generating Station  
Unit 1

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

YEAR

SEQUENTIAL  
NUMBERREVISION  
NUMBER

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Actions:

The I&C technicians and the CO and RO involved with the event were counselled regarding the importance of precise communication.

Similar Occurrences:

Previous similar occurrences relating to failure of communications: LER 85-033; LER 85-006; LER 85-003; LER 84-031.

Previous similar occurrences relating to inadvertent RWCU system isolation on high differential flow: LER 85-003; LER 85-002; LER 84-031.

PHILADELPHIA ELECTRIC COMPANY

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May 29, 1985

Docket No. 50-352

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Washington, DC 20555

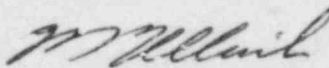
SUBJECT: Licensee Event Report  
Limerick Generating Station - Unit 1

This LER concerns the automatic isolation of the reactor water cleanup system on high differential flow.

Reference:	Docket No. 50-352
Report Number:	85-051
Revision Number:	00
Event Date:	May 7, 1985
Report Date:	May 29, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,



W. T. Ullrich  
Superintendent  
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC  
J. T. Wiggins, Senior Site Inspector  
See Service List

LE22  
1/1



c: Judge Helen F. Hoyt  
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Docket & Service Section (3 Copies)  
James Wiggins  
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January 16, 1985