

Dr. I. Spiewak:  
Institute of Energy Analysis  
P.O. Box 117  
Oak Ridge, TN 37830

Dear Dr. Spiewak:

At the request of DR. K. Araj, I have enclosed a copy of the following IDCOR reports:

- (1) Technical Report 2.1 - Ground Rules for the IDCOR Program,
- (2) Technical Report 3.1 - Define Initial Likely Sequences,
- (3) Technical Report 3.3 - Selection of Dominant Sequences - Update,
- (4) Technical Report 12.1 - Hydrogen Generation During Severe Core Damage Sequences,
- (5) Subtask 12.2 - Hydrogen Distribution in Reactor Containment Buildings (Vol. 1),
- (6) Subtask 12.3, Vol. 1 - Hydrogen Combustion in Reactor Containment Buildings,
- (7) Technical Report 14.1A - Key Phenomenological Models for Assessing Explosive Steam Generation Rates,
- (8) Technical Report 14.1B - Key Phenomenological Models for Assessing Non-Explosive Steam Generation Rates,
- (9) Technical Report 15.1A - In-Vessel Core Melt Progression Phenomena,
- (10) Technical Report 15.2A - Effects of a Hypothetical Core Melt Accident on a PWR Vessel with Top Entry Instruments,
- (11) Technical Report 15.2B - Debris Coolability, Vessel Penetration, and Debris Dispersal,

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(12) Subtask 15.1

- Phenomenological and Modelling Background for the PAR and DRR heatup Codes (Vol. I), and

(13) Subtask 15.3

- Final Report on Core-Concrete Interactions.

Sincerely,

Christopher Ryder  
Accident Source Term Program Office  
Office of Nuclear Regulatory Research  
U.S. Nuclear Regulatory Commission

Enclosures: As stated

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