

October 11, 1996

Mr. Donald A. Reid
Vice President, Operations
Vermont Yankee Nuclear Power Corporation
Ferry Road
Brattleboro, Vermont

SUBJECT: EVALUATION OF FLAW INDICATION FOUND DURING REACTOR PRESSURE VESSEL
INSPECTIONS AT VERMONT YANKEE NUCLEAR POWER STATION (TAC NO.
M96670)

Dear Mr. Reid:

By letter dated October 9, 1996, as supplemented on October 11, 1996, Vermont Yankee Nuclear Power Corporation (the licensee) submitted a flaw evaluation report for NRC review and approval. The report contained the licensee's evaluation of a flaw indication in the reactor pressure vessel (RPV) that exceeded the allowable flaw sizes in IWB-3500 of the American Society of Mechanical Engineers (ASME) Code, Section XI. Ultrasonic examinations (UT) of the RPV were performed in accordance with the requirements of the ASME Code, Section XI, 1986 Edition. The UT examinations resulted in one flaw, out of a total of seven, that was not acceptable by IWB-3500 acceptance standards.

The flaw is located in Plate 1-15, below the circumferential weld that joins Plates 1-12 and 1-15. Plate 1-15 extends into the beltline region; however, the flaw is outside of the core region and is circumferential in orientation. Since the flaw is located near the circumferential weld, it was evaluated by assuming that its location is within the weld metal as well as within the plate. The flaw evaluation was performed using the methodology in Appendix A of Section XI of the ASME Code, and met the criteria in IWB-3600.

Since the licensee's flaw evaluation meets the ASME Code criteria in IWB-3600, Vermont Yankee can be operated without repair to the subject plate. The NRC staff's evaluation is enclosed.

Sincerely,

Original Signed By

C. Craig Harbuck, Acting Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/JI
Office of Nuclear Reactor Regulation

Enclosure: As stated
Docket No. 50-271

cc: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Since the licensee's flaw evaluation meets the ASME Code criteria in IWB-3600, Vermont Yankee can be operated without repair to the subject plate. The NRC staff's evaluation is enclosed.

Sincerely,

C. Craig Harbuck, Acting Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure: As stated

Docket No. 50-271

cc: See next page

D. Reid
Vermont Yankee Nuclear Power
Corporation
cc:

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

R. K. Gad, III
Ropes & Gray
One International Place
Boston, MA 02110-2624

Mr. Richard P. Sedano, Commissioner
Vermont Department of Public Service
120 State Street, 3rd Floor
Montpelier, VT 05602

Public Service Board
State of Vermont
120 State Street
Montpelier, VT 05602

Chairman, Board of Selectmen
Town of Vernon
P.O. Box 116
Vernon, VT 05354-0116

Mr. Jay Thayer, Vice President
Vermont Yankee Nuclear Power
Corporation
Ferry Road
Brattleboro, VT 05301

Mr. Robert J. Wanczyk, Plant Manager
Vermont Yankee Nuclear Power Station
P.O. Box 157, Governor Hunt Road
Vernon, VT 05354

Vermont Yankee Nuclear Power Station

G. Dana Bisbee, Esq.
Deputy Attorney General
33 Capitol Street
Concord, NH 03301-6937

Resident Inspector
Vermont Yankee Nuclear Power Station
U.S. Nuclear Regulatory Commission
P.O. Box 176
Vernon, VT 05354

Chief, Safety Unit
Office of the Attorney General
One Ashburton Place, 19th Floor
Boston, MA 02108

Mr. David Rodham, Director
ATTN: James Muckerheide
Massachusetts Civil Defense Agency
400 Worcester Rd.
P.O. Box 1496
Framingham, MA 01701-0317

Mr. Raymond N. McCandless
Vermont Division of Occupational
and Radiological Health
Administration Building
Montpelier, VT 05602

Mr. J. J. Duffy
Licensing Engineer
Vermont Yankee Nuclear Power
Corporation
580 Main Street
Bolton, MA 01740-1398