



Northern States Power Company

Prairie Island Nuclear Generating Plant

1717 Wakonade Dr. East
Welch, Minnesota 55089

October 7, 1996

10 CFR Part 50
Section 50.73

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Auto-start of No. 11 Component Cooling Water Pump Due to Personnel Error

The Licensee Event Report for this occurrence is attached. In the report, we made one new NRC commitment indicated as the corrective action in italics.

This event was reported via the Emergency Notification System in accordance with 10 CFR Part 50, Section 50.72, on September 7, 1996. Please contact us if you require additional information related to this event.

Michael D Wadley

Michael D Wadley
Plant Manager
Prairie Island Nuclear Generating Plant

c: Regional Administrator - Region III, NRC
NRR Project Manager, NRC
Senior Resident Inspector, NRC
Kris Sanda, State of Minnesota

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Attachment

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Prairie Island Nuclear Generating Plant Unit 1	DOCKET NUMBER (2) 05000 282	PAGE (3) 1 OF 3
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TITLE (4)
Auto-start of No. 11 Component Cooling Water Pump Due to Personnel Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	07	96	96	-- 16 --	0	10	7	96	Prairie Island Unit 2	05000 306
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9) THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)

20.2201(b)	20.2203(a)(2)(v)	50.73(a)(2)(i)	50.73(a)(2)(viii)
20.2203(a)(1)	20.2203(a)(3)(i)	50.73(a)(2)(ii)	50.73(a)(2)(x)
20.2203(a)(2)(i)	20.2203(a)(3)(ii)	50.73(a)(2)(iii)	73.71
20.2203(a)(2)(ii)	20.2203(a)(4)	X 50.73(a)(2)(iv)	OTHER
20.2203(a)(2)(iii)	50.36(c)(1)	50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A
20.2203(a)(2)(iv)	50.36(c)(2)	50.73(a)(2)(vii)	

POWER LEVEL (10)

LICENSEE CONTACT FOR THIS LER (12)

NAME Jack Leveille	TELEPHONE NUMBER (Include Area Code) 612-388-1121
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE).	X NO	EXPECTED SUBMISSION	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 7, 1996, Unit 1 was operating at 100% power. During testing of the residual heat removal pumps per quarterly surveillance procedure, SP-1089, Residual Heat Removal Pumps and Suction Valves from the Refueling Water Storage Tank, the standby component cooling (CC) water pump is started. At the end of the surveillance test the standby CC Pump is shut down. While shutting down the standby CC Pump the operator performing the surveillance test failed to hold the START/STOP switch for 11 CC Pump in the "STOP" position long enough to allow system pressure to stabilize, contrary to the note in the procedure which provided information on ensuring that the system stabilizes prior to releasing the START/STOP switch. As a result, momentary low pressure in the CC system caused 11 CC Pump to automatically restart when the switch was released. The pump was subsequently stopped by following the guidance of the note in the procedure.

(4-95)

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

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Prairie Island Nuclear Generating Plant Unit 1	05000 282	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		96	-- 16 --	0	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION

Surveillance procedure SP-1089, "Residual Heat Removal Pumps and Suction Valves from the Refueling Water Storage Tank," is required by Technical Specification Section 4.5 and ASME Code Section XI. The test consists of a fifteen minute run, on recirculation, of each residual heat removal pump¹ and obtaining pump performance and vibration data. One of the Precautions and Limitations stated in the surveillance procedure is; "Standby CC Pmp could AUTO START on low CC pressure during the RHR Pmp start. The standby CC Pmp should be manually started to preclude this happening".

On September 7, 1996, Unit 1 was operating at 100% power. Routine testing of the Unit 1 residual heat removal pumps was in progress per quarterly surveillance procedure 1089. During this surveillance test the standby component cooling (CC) water pump² is started. At the end of the surveillance test the standby CC pump is shutdown. When the operator attempted to stop 11 CC Pump at 0020:06 as required by SP-1089, he failed to hold the START/STOP switch in the "STOP" position long enough to allow system pressure to stabilize. When the switch was released 11 CC Pump restarted automatically at 0020:09 due to momentary low pressure in the CC system. The operator then recalled that he was required to hold the switch in the "STOP" position long enough to allow the system pressure to stabilize. At 0020:41 11 CC Pump was stopped by holding the START/STOP switch in the "STOP" position until CC system pressure stabilized.

CAUSE OF THE EVENT

The cause of the event was personnel error. The operator overlooked the note preceding the step for stopping the pump. The note provided information on ensuring that the system stabilizes prior to releasing the START/STOP switch to prevent an inadvertent auto start.

ANALYSIS OF THE EVENT

During this event, all equipment operated as designed. The auto restart of the standby component cooling water pump ensured that all safety-related components requiring component cooling water were provided with adequate pressure and flow to perform their related functions; health and safety of the public were unaffected. Since this event involved the automatic start of an engineered safety feature component, it is reportable pursuant to 10CFR50.73(a)(2)(iv).

¹ (EIS System Identifier: BP; EIS Component Identifier: P)

² (EIS System Identifier: CC; EIS Component Identifier: P)

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Prairie Island Nuclear Generating Plant Unit 1	05000 282	96	-- 16 --	0	3 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CORRECTIVE ACTION

Reinforced, with the operator involved in this event, the importance of reading procedure notes before performing the step following a note.

Install "caution" labels on the control boards, providing guidance on stopping a component cooling water pump.

FAILED COMPONENT IDENTIFICATION

There were no failed components.

PREVIOUS SIMILAR EVENTS

Automatic component cooling water pump starts have been reported as Unit 1 LERs 85-007, 87-020 and 89-001, but none of these had similar causes. Unit 1 LER 90-09 involved a auto restart of a CC pump due to momentary low system pressure when stopping a CC pump.