

CONTROL BLOCK

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | C | A | S | 0 | 5 | 2 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

LICENSEE CODE 14 15 LICENSE NUMBER 23 24 LICENSE TYPE 30 31 CAT 32

CONT

01 | REPORT SOURCE 60 61 | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 6 | 1 | 7 | 0 | 3 | 1 | 6 | 8 | 3 | 8 | 0 | 7 | 0 | 5 | 8 | 5 | 9

DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | On 1/26/84, it was determined that on 3/16/83, snubber S2-FW-301-H-003, located

03 | on the 8" demineralized water makeup and transfer line between Condensate

04 | Storage Tanks (CST's), had been discovered frozen.

05 | See Attachment.

06 |

07 |

08 |

09 | SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

W | C | 11 | D | 12 | Z | 13 | S | U | P | O | R | T | 14 | D | 15 | Z | 16

17 | LER/RO REPORT NUMBER 21 22 | 8 | 3 | 23 24 | 1 | 1 | 4 | 3 | 25 26 | 1 | 0 | 3 | 27 28 | 1 | 0 | 3 | 29 30 | 1 | 32

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

G | 18 | Z | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | 0 | N | 23 | N | 24 | N | 25 | P | 0 | 2 | 9 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | It is believed that the snubber was inoperable as a result of internal damage.

11 | The snubber was replaced and declared operable on March 28, 1983. As corrective

12 | action, to prevent untimely engineering evaluations and reportability determina-

13 | tions, administrative controls for dispositioning NCR's have been modified as

14 | discussed in previous LER's including 83-156 (Docket No. 50-361).

15 | FACILITY STATUS 9 10 % POWER 11 12 OTHER STATUS 30 METHOD OF DISCOVERY 31 DISCOVERY DESCRIPTION 32

B | 28 | 0 | 0 | 0 | 29 | NA | 31 | Records Review

16 | ACTIVITY CONTENT RELEASED OF RELEASE 33 AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36

Z | 33 | Z | 34 | NA | NA

17 | PERSONNEL EXPOSURE NUMBER 37 TYPE 38 DESCRIPTION 39

0 | 0 | 0 | 37 | Z | 38 | NA

18 | PERSONNEL INJURIES NUMBER 40 DESCRIPTION 41

0 | 0 | 0 | 40 | NA

19 | LOSS OF OR DAMAGE TO FACILITY TYPE 42 DESCRIPTION 43

Z | 42 | NA

20 | PUBLICITY ISSUED DESCRIPTION 44

N | 44 | NA

NAME OF PREPARER H. E. MORGAN

PHONE (714) 492-7700

8507260114 850705
PDR ADOCK 05000361
S PDR

IE29 1/1

RECEIVED
NRC

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

1985 JUL -8 AM 9:58 **SCE**

REGION V IRE

July 5, 1985

U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

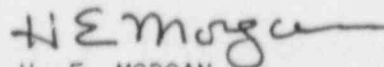
Subject: Docket No. 50-361
Licensee Event Report No. 83-143, Revision 1
San Onofre Nuclear Generating Station, Unit 2

Reference: Letter, J. G. Haynes (SCE) to J. B. Martin (NRC),
dated February 23, 1984; "Licensee Event Report No. 83-143"

The referenced letter provided the required 30-day Licensee Event Report (LER) for an occurrence involving Limiting Condition for Operation (LCO) 3.7.6 associated with snubbers. This revision is being submitted to correct information provided regarding the corrective action taken. The referenced LER-83-156 was erroneously listed as 83-153 and has been corrected on Revision 1.

If you require any additional information, please so advise.

Sincerely,



H. E. MORGAN
STATION MANAGER

Enclosure: LER-83-143, Revision 1

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U.S. Nuclear Regulatory Commission
Document Control Desk

Institute of Nuclear Power Operations (INPO)

11
IE-29

Attachment to LER 83-143

Southern California Edison Company
San Onofre Nuclear Generating Station
Unit No. 2, Docket No. 50-361

Supplemental Information for Event Description and Probable Consequences

The Action Statement associated with LCO 3.7.6 requires that an engineering evaluation be completed on the attached system per Technical Specification 4.7.6.g within 72 hours of the discovery of an inoperable snubber. An engineering evaluation was performed on the attached system September 10, 1983 and showed that thermal loads on the pipe and the CST's were insignificant, and therefore, their operability was unaffected. Public health and safety were not affected.