



Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee 37379-2000

Robert A. Fenech
Vice President, Sequoyah Nuclear Plant

February 16, 1993

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

In the Matter of)	Docket Nos. 50-327
Tennessee Valley Authority)	50-328

SEQUOYAH NUCLEAR PLANT (SQN) - JANUARY 1993 MONTHLY OPERATING REPORT

Enclosed is the January 1993 Monthly Operating Report as required by SQN
Technical Specification 6.9.1.10.

If you have any questions concerning this matter, please call
M. A. Cooper at (615) 843-8924.

Sincerely,

Robert A. Fenech

Enclosure
cc: See page 2

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U.S. Nuclear Regulatory Commission

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cc (Enclosure):

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TENNESSEE VALLEY AUTHORITY

NUCLEAR POWER GROUP
SEQUOYAH NUCLEAR PLANT

MONTHLY OPEATING REPORT
TO THE
NUCLEAR REGULATORY COMMISSION
JANUARY 1993

UNIT 1

DOCKET NUMBER 50-327

LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

OPERATIONAL SUMMARY
JANUARY 1993

UNIT 1

Unit 1 generated 762,990 megawatthours (MWh) (gross) electrical power during January with a capacity factor of 88.26 percent.

Unit 1 was in Mode 3 at the beginning of January after the unplanned reactor trip on December 31, 1992. Unit 1 reactor was taken critical on January 3 at 0152 EST and was online at 1039 EST on January 4. Unit 1 reached 100 percent reactor power at 0135 EST on January 6 and operated at approximately 100 percent reactor power through the end of January.

UNIT 2

Unit 2 generated 506,475 MWh (gross) electrical power during January with a capacity factor of 58.58 percent.

Unit 2 remained in Mode 3 at the beginning of January after the unplanned reactor trip on December 31, 1992. Unit 2 was taken critical on January 2 at 1447 EST and was at 13 percent reactor power on January 3 at 0713 EST. On January 4 at 0853 EST, the decision was made to remove the "B" phase generator bushing and replace it before returning Unit 2 to service. Unit 2 reactor power level was reduced to 8 percent.

On January 7 at 0050 EST, a Unit 2 shutdown was initiated to allow further investigation of the control circuit on the centrifugal charging pump suction valves. Unit 2 entered Mode 3 at 0105 EST.

Unit 2 was taken critical on January 9 at 1808 EST, and was returned to service on January 10 at 0256 EST. Unit 2 reached 100 percent power on January 13 at 2230 EST.

On January 28 at 1200 EST, a Unit 2 reactor power decrease was initiated to repair a leak on an extraction steam line. On January 29 at 0416 EST, with reactor power at approximately 30 percent, the decision was made to take Unit 2 to Mode 3 to allow permanent pipe repair of the steam leak and the repair of a main generator hydrogen leak.

Unit 2 was removed from service at 0645 EST on January 29 and entered Mode 3. After maintenance was completed, Unit 2 was taken critical at 1753 EST on January 30 and was online at 1204 EST on January 31. Unit 2 was at 43 percent reactor power at the end of January.

POWER-OPERATED RELIEF VALVES (PORVs) AND SAFETY VALVES SUMMARY

There were no challenges to PORVs or safety valves in January.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327 UNIT No. One DATE: 01-02-93
 COMPLETED BY: T. J. Hollomon TELEPHONE: (615) 843-7528
 MONTH: JANUARY 1993

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-29	17	1153
2	-32	18	1150
3	-29	19	1152
4	87	20	1153
5	651	21	1152
6	1137	22	1154
7	1139	23	1154
8	1139	24	1154
9	1139	25	1153
10	1146	26	1152
11	1151	27	1152
12	1152	28	1149
13	1152	29	1144
14	1151	30	1145
15	1152	31	1149
16	1152		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328 UNIT No. Two DATE: 01-02-93
 COMPLETED BY: T. J. Hollomon TELEPHONE: (615) 843-7528
 MONTH: JANUARY 1993

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-29	17	1135
2	-32	18	1135
3	-29	19	1135
4	-27	20	1135
5	-16	21	1135
6	-18	22	1135
7	-15	23	1135
8	-19	24	1134
9	-15	25	1133
10	294	26	1132
11	1066	27	1132
12	1089	28	945
13	1099	29	71
14	1132	30	-16
15	1141	31	104
16	1133		

DOCKET NO. 50-327
DATE 02/08/93
COMPLETED BY I. J. Holloman
TELEPHONE (615) 843-7528

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	744	744	101,593
12. Number of Hours Reactor Was Critical	694.1	694.1	55,442
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	661.4	661.4	54,269.3
15. Unit Reserve Shutdown Hours	0.0	0	0
16. Gross Thermal Energy Generated (MWH)	2,195,599.6	2,195,599.6	177,173,444
17. Gross Electrical Energy Generated (MWH)	762,990	762,990	60,107,104
18. Net Electrical Energy Generated (MWH)	733,348	733,348	57,655,570
19. Unit Service Factor	88.9	88.9	52.4
20. Unit Availability Factor	88.9	88.9	53.4
21. Unit Capacity Factor (Using MDC Net)	87.9	87.9	50.6
22. Unit Capacity Factor (Using DER Net)	85.9	85.9	49.4
23. Unit Forced Outage Rate	11.1	11.1	38.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
The Cycle 6 refueling outage is scheduled to begin on April 16, 1993, with a duration of 65 days.
25. If Shut Down At End Of Report Period, Estimated Date of Startup:

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE 02/08/93
COMPLETED BY T. J. Hollomon
TELEPHONE (615) 843-7528

OPERATING STATUS

- | | Notes |
|---|-------|
| 1. Unit Name: <u>Sequoyah Unit Two</u> | |
| 2. Reporting Period: <u>January 1993</u> | |
| 3. Licensed Thermal Power (Mwt): <u>3411.0</u> | |
| 4. Nameplate Rating (Gross MWe): <u>1220.6</u> | |
| 5. Design Electrical Rating (Net MWe): <u>1148.0</u> | |
| 6. Maximum Dependable Capacity (Gross MWe): <u>1162.0</u> | |
| 7. Maximum Dependable Capacity (Net MWe): <u>1122.0</u> | |
| 8. If Changes Occur in Capacity Ratings (Item Numbers 3 Through 7) Since Last Report, Give Reasons: | |

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	<u>744</u>	<u>744</u>	<u>93,553</u>
12. Number of Hours Reactor Was Critical	<u>605.5</u>	<u>605.5</u>	<u>56,819</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>471.8</u>	<u>471.8</u>	<u>55,549.2</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,516,519.5</u>	<u>1,516,519.5</u>	<u>174,698,299</u>
17. Gross Electrical Energy Generated (MWH)	<u>506,475</u>	<u>506,475</u>	<u>59,229,594</u>
18. Net Electrical Energy Generated (MWH)	<u>482,998</u>	<u>482,998</u>	<u>56,698,273</u>
19. Unit Service Factor	<u>63.4</u>	<u>63.4</u>	<u>59.4</u>
20. Unit Availability Factor	<u>63.4</u>	<u>63.4</u>	<u>59.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>57.9</u>	<u>57.9</u>	<u>54.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>56.5</u>	<u>56.5</u>	<u>52.8</u>
23. Unit Forced Outage Rate	<u>36.6</u>	<u>36.6</u>	<u>33.2</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: JANUARY 1993DOCKET NO: 50-327UNIT NAME: OneDATE: 02/08/93COMPLETED BY: T. J. HollomanTELEPHONE: (615) 843-7528

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action to Prevent Recurrence
									Unit 1 was in Mode 3 at the beginning of January. Unit 1 was returned to service on January 4.

¹F: Forced
S: Scheduled

²Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training and License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Existing Outage
5-Reduction
9-Other

⁴Exhibit G-Instructions
for Preparation of Data
Entry sheets for Licensee
Event Report (LER) File
(NUREG-1022)

⁵Exhibit I-Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: JANUARY 1993DOCKET NO: 50-328UNIT NAME: TwoDATE: 02/08/93COMPLETED BY: T. J. HollomanTELEPHONE: (615) 843-7528

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action to Prevent Recurrence
12	930128	F	53.3	B	1	N/A	SB TK	PSF PSF	<p>Unit 2 was in Mode 3 at the beginning of January. On January 2, Unit 2 was taken critical and reached 13 percent power on January 3. On January 7 at 0050 EST, a Unit 2 shutdown was initiated to allow investigation of the control circuitry on the centrifugal charging pump suction valves. The control circuitry was determined to be correct. Unit 2 entered Mode 3 at 0105 EST.</p> <p>On January 28 at 1200, a power decrease was initiated to repair a leak on an extraction steam line. Unit 2 was removed from service at 0645 EST on January 29 and entered Mode 3 for maintenance on the A2 MSR steam leak and repair of a main generator hydrogen leak. The extraction steam line leak was the result of erosion and the affected segments of piping were replaced. The generator hydrogen leak was caused by a failure of a gasket at a flange connection. The gasket was replaced and the system was returned to service. Unit 2 was returned to service on January 31.</p>

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S: Scheduled

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A-Equipment Failure (Explain)
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