

## LICENSEE EVENT REPORT (LER)

|  |  |                                      |                      |
|--|--|--------------------------------------|----------------------|
| FACILITY NAME (1)<br>Browns Ferry - Unit 3 |  | DOCKET NUMBER (2)<br>0 5 0 0 0 2 9 6 | PAGE (3)<br>1 OF 0 2 |
|--|--|--------------------------------------|----------------------|

TITLE (4)  
Inadvertent Initiation of Containment Isolation System

| EVENT DATE (5)        |     |      | LER NUMBER (6) |                   |                 | REPORT DATE (7) |     |      | OTHER FACILITIES INVOLVED (8) |   |                  |   |   |   |   |   |   |   |
|-----------------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---|------------------|---|---|---|---|---|---|---|
| MONTH                 | DAY | YEAR | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH           | DAY | YEAR | FACILITY NAMES                |   | DOCKET NUMBER(S) |   |   |   |   |   |   |   |
| 0                     | 4   | 2    | 1              | 8                 | 5               | 8               | 5   | 0    | 1                             | 0 | 0                | 5 | 0 | 0 | 0 | 2 | 5 | 9 |
| Browns Ferry - Unit 1 |     |      |                |                   |                 |                 |     |      |                               |   | 0 5 0 0 0 2 5 9  |   |   |   |   |   |   |   |
| Browns Ferry - Unit 2 |     |      |                |                   |                 |                 |     |      |                               |   | 0 5 0 0 0 2 6 0  |   |   |   |   |   |   |   |

|                           |                   |  |   |                      |  |  |  |  |  |  |  |
|---------------------------|-------------------|--|---|----------------------|--|--|--|--|--|--|--|
| OPERATING MODE (9)<br>N   |                   | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11) |   |                      |  |  |  |  |  |  |  |
| POWER LEVEL (10)<br>0 0 0 | 20.402(b)         | 20.405(c)  | X | 50.73(a)(2)(iv)      | 73.71(b)   |  |  |  |  |  |  |
|                           | 20.405(a)(1)(i)   | 50.36(c)(1)  |   | 50.73(a)(2)(v)       | 73.71(c)   |  |  |  |  |  |  |
|                           | 20.405(a)(1)(ii)  | 50.36(c)(2)  |   | 50.73(a)(2)(vii)     | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |  |  |  |  |  |  |
|                           | 20.405(a)(1)(iii) | 50.73(a)(2)(i)   |   | 50.73(a)(2)(viii)(A) |  |  |  |  |  |  |  |
|                           | 20.405(a)(1)(iv)  | 50.73(a)(2)(ii)  |   | 50.73(a)(2)(viii)(B) |  |  |  |  |  |  |  |
|                           | 20.405(a)(1)(v)   | 50.73(a)(2)(iii)   |   | 50.73(a)(2)(ix)      |  |  |  |  |  |  |  |

|                                    |  |  |  |  |  |  |  |  |  |                     |  |  |  |
|------------------------------------|--|--|--|--|--|--|--|--|--|---------------------|--|--|--|
| LICENSEE CONTACT FOR THIS LER (12) |  |  |  |  |  |  |  |  |  | TELEPHONE NUMBER    |  |  |  |
| NAME<br>Pat Ebersole               |  |  |  |  |  |  |  |  |  | AREA CODE<br>2 0 5  |  |  |  |
|                                    |  |  |  |  |  |  |  |  |  | 7 2 9 1 - 1 3 7 8 8 |  |  |  |

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) |        |           |              |                   |       |        |           |              |                   |
|--|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|
| CAUSE  | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |
|  |        |           |              |                   |       |        |           |              |                   |
|  |        |           |              |                   |       |        |           |              |                   |
|  |        |           |              |                   |       |        |           |              |                   |

|  |  |  |  |  |                               |       |     |      |
|--|--|--|--|--|-------------------------------|-------|-----|------|
| SUPPLEMENTAL REPORT EXPECTED (14)                    |  |  |  |  | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
| YES (If yes, complete EXPECTED SUBMISSION DATE) X NO |  |  |  |  |                               |       |     |      |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

During placement of an equipment hold order, an electrical short was caused when the fuse that was being removed shorted against the adjacent fuse clip. The electrical short resulted in an inadvertent initiation of several systems associated with a group VI (ventilation) primary containment isolation signal.

The availability of affected safety systems to perform required functions was not reduced by this event.

After verifying that initiation of the safety systems were a result of the short caused during fuse removal, all affected safety systems were returned to standby readiness by operations personnel.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104  
EXPIRES: 8/31/85

| FACILITY NAME (1)                | DOCKET NUMBER (2)   | LER NUMBER (6) |                   |                 | PAGE (3) |        |
|----------------------------------|---------------------|----------------|-------------------|-----------------|----------|--------|
|                                  |                     | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER |          |        |
| Browns Ferry - Units 1, 2, and 3 | 0 5 0 0 0 2 9 6 8 5 | -0             | 1 0               | -0              | 0 0 2    | OF 0 2 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On April 21, 1985 at 2230, a short caused during fuse removal for placement of an equipment hold order resulted in inadvertent initiation of several safety systems associated with a group VI (ventilation) primary containment isolation (JM) signal. At the time of this event, units 1 and 3 were in a cold shutdown condition, and unit 2 was in a refueling outage with all fuel removed from the vessel. This event primarily affected unit 3, but units 1 and 2 were affected in that the refuel zone ventilation (VA) is common to all three units.

Systems that inadvertently started as a result of the fuse short were:

1. Reactor and Refuel Zone Ventilation Systems (VA)
2. Isolation of Drywell Containment Air Monitor (VB)
3. Isolation of Primary Containment Hydrogen and Oxygen Analyzers (VA)
4. Initiation of the Standby Gas Treatment System (BH)
5. Initiation of the Control Room Emergency Ventilation Unit (VI)

The root cause of this event is attributed to personnel error during fuse removal. It should be pointed out, however, that fuse removal required working in an electrical cabinet that involved very close quarters, and the adjacent fuse clip was extremely close. To ensure that occurrences of this type of event are minimized as much as possible, operations personnel shall be instructed to exercise care during fuse removal. This will be discussed with all licensed personnel during operator supplemental training.

The availability of affected safety systems to perform their required function was not reduced by this event.

After verifying that the initiation of the safety systems was a result of the electrical short caused during fuse removal, all affected safety systems were returned to standby readiness by operations personnel.

Responsible Plant Section - OP

Previous Events - BFRO-50-259/84023; 50-260/82025

TENNESSEE VALLEY AUTHORITY  
Browns Ferry Nuclear Plant  
P. O. Box 2000  
Decatur, Alabama 35602

May 17, 1985

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

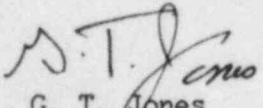
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 3 -  
DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE  
OCCURRENCE REPORT BFRO-50-296/85010

The enclosed report provides details concerning the inadvertent  
initiation of containment isolation system. This report is submitted  
in accordance with 10 CFR 50.73 (a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
G. T. Jones  
Plant Manager  
Browns Ferry Nuclear Plant

BCM:PNE:BDL

Enclosures

cc (Enclosures):

Regional Administrator  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, Georgia 30303

INPO Records Center  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Resident Inspector, BFN

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11