



Integrated Nuclear Services

JHT/96-67
October 9, 1996

Project No. 693

Mr. David B. Matthews, Chief
Generic Issues and Environmental Projects Branch
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Subject: Topical Report Prioritization

Dear Mr. Matthews:

This letter is in response to your September 17, 1996 letter on topical report prioritization. Separate tables are attached for FTI topical reports and B&WOG topical reports.

If you have any questions on the tables, please call me at 804-832-2817 or Mr. Robert Borsum at 301-230-2100.

Very truly yours,

James H. Taylor, Manager
Licensing Services

JHT/bcc

Attachments

c: U.S. NRC Document Control Desk
J. L. Birmingham, PGEB/DRPM/NRR/NRC
R. B. Borsum-FTI/MD82

9610150237 961009
PDR TOPRP EMVBW
C PDR

1000000000

3315 Old Forest Road, P.O. Box 10935, Lynchburg, VA 24506-0935
Telephone: 804-832-3000 Fax: 804-832-3663

TOTO
1/1

RD-8-2 Framatome
X RD-8-2 B&W

TDP

B&WOG TOPICAL REPORTS

| REPORT | REPORT TITLE | DATE OF SUBMITTAL | PRIORITY | REMARKS |
|--------------------------------|---|-----------------------------|----------|---|
| BAW-1543 Rev. 4, Supp. 2 | Master Integrated Reactor Vessel Material Surveillance Program | 8/2/96 | Normal | |
| BAW-2086 | J-R Test Procedure for Wedge-Opening Loading Fracture Toughness Specimens | 6/30/89 Rev. 1, 12/31/93 | Normal | |
| BAW-2181 | CRAFT2-RELAP-MOD2 Comparison for Small Break LOCA Analysis | 2/19/93 | Low | |
| BAW-2244 | Demonstration of the Management of Aging Effects for the Pressurizer | 8/31/95 | High | To support license renewal program. DSE issued, review almost complete |
| BAW-2245 Rev. 1 | Initial RT _{NDT} of LINDE 80 Welds Based on Fracture Toughness in the Transition Range | 9/22/95 | Defer | On hold pending ASTM Committee Action |
| BAW-2251 | Demonstration of the Management of Aging Effects for the Reactor Vessel | 7/31/96 | High | To support license renewal program. Incorporates BAW-2274, Fracture Mechanics Analysis of Postulated Underclad Cracks in B&W Designed Reactor Vessels for the Period of Extended Operation; BAW-2275, Low Upper-Shelf Toughness Fracture Mechanics Analysis of B&W Designed Reactor Vessels for 48 EFPY |
| BAW- 10167P Supp. 3 | Justification for Increasing the Reactor Trip System On-Line Test Intervals | 6/7/96 | High | Review almost complete. To support ANO-1 & Davis Besse licensing actions. |
| BAW-10179 Rev. 2 | Safety Criteria and Methodology for Acceptable Cycle Reload Analyses | early-1997 | High | Required for reload analysis. Much of the information is in previously reviewed reports. |
| BAW-XXXX | Fluence and Uncertainty Analysis | early-1997 | Normal | |
| BAW-XXXX | Steam Generator Tube Alternate Repair Criteria | late-1997 | Normal | |

October 1, 1996

FTI TOPICAL REPORTS

| REPORT | REPORT TITLE | DATE OF SUBMITTAL | PRIORITY | REMARKS |
|--------------------------|--|-----------------------------|----------|---|
| BAW-10159P | BWCMV Applicability to V5H Fuel | 4/24/95 | High | Needed to support Sequoyah 1 Cycle 9 5/97. |
| BAW-10164 | RELAP5-MOD2-B&W Advanced Computer Program for LWR LOCA & Non-LOCA Transient Analysis | 12/28/87 | Normal | |
| BAW-10180 | NEMO Rod Worth Uncertainty for Westinghouse 193 FA Plants | 9/27/96 | High | Needed for Sequoyah 1 Cycle 9 5/97. |
| BAW-10186 | Extended Burnup Evaluation (BWFC) | 11/24/92 | High | Review almost complete. |
| BAW-10192P | BWNT Loss-of-Coolant Accident Evaluation Model for Once-Through Steam Generator Plants | 2/15/94 | High | Review almost complete. |
| BAW-10193 | RELAP5/MOD2 - B&W Safety Analysis for B&W-Designed Pressurized Water Reactors | 8/14/95 | High | Needed for future transient analyses and power updates. |
| BAW-10199P Addendum 1 | BWU Applicability to Mark B11 and Mark-BW with Mid-Span Mixing Grids | 9/30/96 | High | BWU applicability to Mark-B11 is needed for safety analyses that Duke is scheduled to begin in 1/97. BWU applicability to Mark-BW needed for North Anna LTAs for insertion in 5/97. |
| BAW-10219P | Electrosleeving Qualification of PWR Recirculating Steam Generator Tube Repair | 12/11/95 Rev. 1, 3/26/96 | High | Required to support licensing actions by Callaway, Calvert Cliffs and South Texas Project. |
| BAW-10220P | Mark-BW Fuel Assembly Application for Sequoyah Units 1&2 | 3/5/96 | High | Required to support reload of Sequoyah 5/97. |
| BAW-10221 | NEMO-K | 9/30/96 | High | Needed by 6/97 to enhance capability to analyze RIA transients in. |
| BAW-102AA | Alloy 5 Design Report | Scheduled 7/97 | High | SER requested 7/98. This approval is required to support rule change to include Alloy 5 in 10CFR50.44 and 10CFR50.46 and to support full batch implementation in 4th quarter 1998. |

October 9, 1996

FTI TOPICAL REPORTS

| REPORT | REPORT TITLE | DATE OF SUBMITTAL | PRIORITY | REMARKS |
|-----------|---|-------------------|----------|---|
| BAW-102BB | SCIENCE | Scheduled 9/97 | Normal | SER requested 9/97. SCIENCE will be standard neutronics system code for Framatome. |
| BAW-102CC | Mark-B11 Design Report | Scheduled 6/97 | Normal | Approval requested 6/98. Supports full batch implementation of Mark-B11 fuel for Oconee units. |
| BAW-102DD | Faulted Condition Topical | Scheduled 1/97 | Normal | SER requested 6/98. Serves as Framatome standard methodology for FA structural analyses. |
| BAW-102EE | COPERNIC | Scheduled 1/98 | Normal | Approval requested 1/99. Serves as Framatome standard for fuel rod thermal performance analyses. |
| BAW-102FF | Framatome Standard Fuel Assembly Design | Scheduled 9/98 | Normal | Approval requested 4/99. Provides description and analyses of Framatome standard 17x17 FA. |
| BAW-102GG | AFA3G15 Fuel Assembly Design Report | Scheduled 6/99 | Normal | SER requested 6/00. Provides descriptions and analyses of Framatome standard 15x15 FA. |
| BAW-102HH | Framatome Standard CHF Correlation | Scheduled 9/98 | Normal | SER requested 9/99. Describes development and test results of Framatome standard CHF correlation. |
| | | | | |
| | | | | |
| | | | | |

October 9, 1996