



Commonwealth Edison  
LaSalle County Nuclear Station  
2601 N. 21st. Rd.  
Marseilles, Illinois 61341  
Telephone 815/357-6731

February 11, 1993


Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Mail Station PL-137  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for January 1993.

Very truly yours

  
Gary F. Spedl  
Station Manager  
LaSalle County Station

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JANUARY 1993

COMMONWEALTH EDISON COMPANY

ERC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor sub-critical, Generator off-line, Refuel outage (L1R05) in progress.
23	1354	Reactor critical.
26	2320	Manual reactor scram for a safety relief valve sticking in the open position.
28	0900	Reactor critical.
30	1300	Generator on-line at 60 Mwe.
	1510	Manual turbine trip due to high bearing vibrations.
	2120	Generator on-line at 60 Mwe.
31	0010	Manual turbine trip due to high bearing vibrations.
	0720	Generator on-line at 60 Mwe.
	1710	Manual turbine trip due to high bearing vibrations.
	2400	Reactor critical, Generator off-line, forced outage in progress.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

Established Technical Specification requirements for the steam discharge volume vent and drain valves.

Revised the Technical Specification bases for the control rod housing support.

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports ).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
93-001-00	01/05/93	Instrument spike which resulted in a Division II low pressure core injection pump start and a 1A diesel generator start.
93-002-00	01/26/93	1B21-F013A failed to close during startup testing.
93-003-00	01/29/93	Failure of the reactor core isolation cooling barometric condenser pump.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)



C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L06691	250 Volt Battery Charger IDC02E	Charger failure board	Inaccurate output voltage indication	Replaced Failure Board
L11120	Reactor Recirc flow summer 1B33-K507A	Test card	Upscale trip setpoint failed to repeat	Replaced test card
L13610	Reactor Core Isolation Cooling min. flow valve 1E51-F019	Limiter torque motor	Valve failed to operate	Replaced motor
L14731	MSIV Leakage Control pressure transmitter PT-1E32-NC01A	Vent valve leakage	None	Replaced valve
L15237	Reactor Core Isolation Cooling barometric cond. vacuum pump 1E51-C005	Pump shaft seal leak	None	Adjusted packing.
L15545	High Pressure Core Spray inj. valve 1E22-F004	Valve leakage	None	Replaced gasket.
L16598	Reactor Core Isolation Cooling barometric cond. pump stop valve 1E51-F004	Air regulator leaking	None	Replaced air regulator.
L17251	Control Rod Drive Hydraulic Control Unit 54-27	Scram water accumulator leaking	None	Replaced accumulator.

C. TABLE 1 (Unit 1) --CONTINUED  
MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L18211	MSIV Leakage Control isolation valve 1E32-F001E	Valve leaking	Valve failed local leak rate testing	Lapped valve disc and repacked valve.
L18256	MSIV Leakage Control steam tunnel discharge valve 1E32-F008	Thermal overloads	Unable to remotely open valve	Replaced thermal overloads.
L18382	Reactor Core Isolation Cooling isolation valve 1E51-F064	Valve leaking	Valve failed local leak rate testing	Lapped valve disc and replaced gasket.
L18954	Reactor Core Isolation Cooling turbine exhaust isolation valve 1E51-F086	High valve packing loads	Unable to test valve	Replaced packing.
L18960	Reactor Core Isolation Cooling turbine exhaust stop valve 1E51-F080	High valve packing loads	Unable to test valve	Replaced packing.
L19154	Control Rod Drive Hydraulic Control Unit 02-27	Directional control valves	Rod bypassed	Replace control valves.
L19217	Main Steam Isol. Valve 1B21-F022B	Air control valve leaking	None	Replaced air control valve
L19433	Control Rod Drive Hydraulic Control Unit 18-15	Scram water accumulator leaking	Unable to maintain pressure	Replaced accumulator.



C. TABLE 1 (Unit 1) —CONTINUED  
MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L19506	Channel C Intermediate Range Monitor 1C51-K601C	Pre-amplifier	None	Replaced pre-amplifier
L19522	Channel R Intermediate Range Monitor recorder 1C51-R603B	Selector switch	Erratic indication	Replaced selector switch.
L19534	Feedwater check valve 1B21-F032A	Solenoid leaking	None	Replaced solenoid
L19563	Reactor Recirc. isolation valve 1B33-F345B	Limit switch	Inaccurate valve indication	Replaced limit switch.
L19701	Hydrogen Recomb. flow indication	Flow indicator 1FI-HG022	Inaccurate flow indication	Replaced flow indicator
L19983	Switchgear Heat Removal Recirc damper 1VX08Y	Damper actuator	Damper failed to open	Replaced actuator.
L20093	Reactor Core Iso. Cooling barometric condenser vacuum pump 1E51-C005	Motor brushes frayed	None	Replaced motor brushes.
L20365	Control Rod Drive Hydraulic Control Unit 30-11	Directional control valve	None	Replaced the directional valve.

(No SOR failures this month)

TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE February 10, 1993  
COMPLETED BY M.J.CIALKOWSKI  
TELEPHONE (815) 357-6761

OPERATING STATUS

- |  |              |
|--|--------------|
| 1. REPORTING PERIOD:                                   | January 1993 |
| GROSS HOURS IN REPORTING PERIOD:                       | 144          |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (M/t):             | 3,323        |
| MAX DEPENDABLE CAPACITY (MWe-Net):                     | 1,036        |
| DESIGN ELECTRICAL RATING (MWe-Net):                    | 1,078        |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): | N/A          |
| 4. REASON FOR RESTRICTION (IF ANY):                    |              |

	THIS MONTH	YEAR TO DATE	CUMULATIVE
	-----	-----	-----
5. REACTOR CRITICAL TIME (HOURS)	168.4	168.4	54,093.2
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	14.8	14.8	52,922.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh)	36,968	36,968	155,097,126
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	2,142	2,142	51,794,491
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	-6,846	-6,846	49,641,244
12. REACTOR SERVICE FACTOR (%)	22.6	22.6	67.9
13. REACTOR AVAILABILITY FACTOR (%)	22.6	22.6	69.9
14. UNIT SERVICE FACTOR (%)	2.0	2.0	66.4
15. UNIT AVAILABILITY FACTOR (%)	2.0	2.0	66.4
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-0.9	-0.9	60.1
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	-0.9	-0.9	57.8
18. UNIT FORCED OUTAGE FACTOR (%)	57.7	57.7	6.9

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
No Outages Scheduled

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 02/01/93

TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE February 10, 1993  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-5761

REPORT PERIOD: January 1993

DAY	POWER	DAY	POWER
-----	-----	-----	-----
1	-12	17	-12
2	-12	18	-12
3	-12	19	-12
4	-12	20	-12
5	-12	21	-12
6	-12	22	-12
7	-12	23	-12
8	-12	24	-12
9	-12	25	-12
10	-12	26	-12
11	-12	27	-12
12	-12	28	-12
13	-12	29	-12
14	-12	30	9
15	-12	31	53
16	-12		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS : 20%  
(UNIT 1)

YEARLY SEQUENTIAL NUMBER	DATE {YYMMDD}	TYPE F: FORCED S: SCHEDULED	DURATION {HOURS}	REASCS	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
1	921003	S	541.9	C	1	Refuel outage (LIR05)
2	930130	F	6.2	A	4	Manual turbine trip due to high bearing vibrations (the reactor remained critical)
3	930131	F	7.2	A	4	Manual turbine trip due to high bearing vibrations (the reactor remained critical)
4	930131	F	6.8	A	4	Manual turbine trip due to high bearing vibrations (the reactor remained critical)

## SUMMARY OF OPERATION .

The unit entered a scheduled refuel outage on 10/03/92. The unit was returned to service 01/30/93.  
The unit experienced three forced outages due to high bearing vibrations caused by the newly installed  
main turbine low pressure rotors.

F. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief valve operations

Date	Valves Actuated	Type of Actuation	Plant Condition	Description of Event
01/26/93	1B21-F013A	Manual	Startup	Startup testing (LER # 93-002-C0)
01/26/93	1B21-F013C	Manual	Startup	Startup testing
01/26/93	1B21-F013D	Manual	Startup	Startup testing
01/26/93	1B21-F013E	Manual	Startup	Startup testing
01/26/93	1B21-F013K	Manual	Startup	Startup testing
01/26/93	1B21-F013F	Manual	Startup	Startup testing
01/26/93	1B21-F013R	Manual	Startup	Startup testing
01/26/93	1B21-F013S	Manual	Startup	Startup testing
01/26/93	1B21-F013U	Manual	Startup	Startup testing
01/26/93	1B21-F013V	Manual	Startup	Startup testing
01/28/93	1B21-F013A	Manual	Startup	Startup testing
01/28/93	1B21-F013B	Manual	Startup	Startup testing
01/28/93	1B21-F013D	Manual	Startup	Startup testing
01/28/93	1B21-F013F	Manual	Startup	Startup testing
01/28/93	1B21-F013G	Manual	Startup	Startup testing
01/28/93	1B21-F013H	Manual	Startup	Startup testing
01/28/93	1B21-F013J	Manual	Startup	Startup testing
01/28/93	1B21-F013L	Manual	Startup	Startup testing
01/28/93	1B21-F013M	Manual	Startup	Startup testing
01/28/93	1B21-F013N	Manual	Startup	Startup testing
01/28/93	1B21-F013P	Manual	Startup	Startup testing
01/28/93	1B21-F013V	Manual	Startup	Startup testing

2. ECCS System Outages  
(See Table 5)

3. Changes to the Off-Site Dose Calculation Manual  
(None)

4. Major Changes to Radioactive Waste Treatment Systems  
(None)

5. Indications of Failed Fuel Elements  
(None)

(UNIT 1)

Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

OUTAGE NO.	EQUIPMENT	PURPOSE
(U-0)		
0005	0DG08CA	Inspection
0006	0DG08CB	Inspection
(U-1)		
0012	1E51-F008	Administrative control
0019	1E51-F013	Logic testing
0024	1DG035	Limiter torque spring pack replacement
0028	1E12-F087B	Administrative control
0044	1DG08CB	Air leak repairs
0045	1E12-F024B	Administrative control
0047	1E51-F064	Administrative control
0050	1E12-F004C	Thermal overload replacement
0051	1E21-F034	Administrative control
0060	1E51-C005	Motor inspection
0123	1E51-C002	Inspection
0144	1DG01K	Lubrication
0155	1E51-C004	Pump motor replacement



LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JANUARY 1993

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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E. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

F. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Safety Relief Valve Operations
2. ECCS System Outages
3. Off-Site Dose Calculation Manual Changes
4. Major Changes to Radioactive Waste Treatment System
5. Indications of Failed Fuel Elements

## I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 1133 Mwe.
	0300	Reduced power level to 1000 Mwe due to system load.
	2000	Increased power level to 1110 Mwe.
2	0230	Reduced power level to 850 Mwe due to system load.
	1130	Increased power level to 1135 Mwe.
3	0300	Reduced power level to 850 Mwe due to system load.
	1800	Increased power level to 1135 Mwe.
4	0000	Reduced power level to 845 Mwe due to system load.
	1300	Increased power level to 1135 Mwe.
5	0200	Reduced power level to 845 Mwe due to system load.
	1400	Increased power level to 1135 Mwe.
6	0100	Reduced power level to 845 Mwe due to system load.
	1200	Increased power level to 1135 Mwe.
8	0200	Reduced power level to 995 Mwe due to system load.
	1000	Increased power level to 1140 Mwe.
10	0200	Reduced power level to 845 Mwe due to system load.
	1100	Increased power level to 1135 Mwe.
12	0200	Reduced power level to 895 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
13	0030	Reduced power level to 1000 Mwe to perform monthly surveillances and CRD exercising.
	1100	Increased power level to 1135 Mwe.
15	0400	Reduced power level to 945 Mwe due to system load.
	1300	Increased power level to 1135 Mwe.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
16	0330	Reduced power level to 1000 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
17	0330	Reduced power level to 995 Mwe due to system load.
	0900	Increased power level to 1140 Mwe.
19	0200	Reduced power level to 1040 Mwe due to system load.
	0700	Increased power level to 1140 Mwe.
21	0230	Reduced power level to 995 Mwe due to system load.
	1000	Increased power level to 1140 Mwe.
	2330	Reduced power level to 1040 Mwe due to system load.
22	0900	Increased power level to 1135 Mwe.
24	0200	Reduced power level to 850 Mwe due to system load.
	1300	Increased power level to 1135 Mwe.
26	0000	Reduced power level to 1000 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
27	0130	Reduced power level to 1000 Mwe due to system load.
	0800	Increased power level to 1135 Mwe.
31	0130	Reduced power level to 995 Mwe due to system load.
	0830	Increased power level to 1135 Mwe.
	2400	Reactor critical, Generator on-line at 1075 Mwe due to system load.

- B. AMMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION  
Established Technical Specification requirements for the scram discharge volume vent and drain valves.

Revised the Technical Specification bases for the control rod housing support.

- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOW differential pressure switch failure reports ).  
(See Table 1)

- D. LICENSEE EVENT REPORTS (Unit 2)

(None)

- E. DATA TABULATIONS (Unit 2)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)



C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L19577	High Pressure Core Spray Diesel Generator output breaker	Breaker spring charging motor	None	Replaced charging motor.
L19801	Control Room HVAC ammonia detector CXY-VC165A	Drive motor	Spurious alarms	Replaced drive motor.

(No SCR failures this month.)

TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LASALLE TWO  
DATE February 10, 1993  
COMPLETED BY M.J.CIALKOWSKI  
TELEPHONE (815) 357-6761

OPERATING STATUS

- |  |              |
|--|--------------|
| 1. REPORTING PERIOD:                                   | January 1993 |
| GROSS HOURS IN REPORTING PERIOD:                       | 744          |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):             | 3,323        |
| MAX DEPENDABLE CAPACITY (MWe-Net):                     | 1,036        |
| DESIGN ELECTRICAL RATING (MWe-Net):                    | 1,078        |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): | N/A          |
| 4. REASON FOR RESTRICTION (IF ANY):                    |              |

	THIS MONTH	YEAR TO DATE	CUMULATIVE
	-----	-----	-----
5. REACTOR CRITICAL TIME (HOURS)	744.0	744.0	51,757.5
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	744.0	50,800.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh <sub>t</sub> )	2,402,397	2,402,397	152,417,284
10. ELECTRICAL ENERGY GENERATED (MWh <sub>e</sub> -Gross)	819,637	819,637	50,720,946
11. ELECTRICAL ENERGY GENERATED (MWh <sub>e</sub> -Net)	792,884	792,884	48,705,817
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	71.2
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	73.6
14. UNIT SERVICE FACTOR (%)	100.0	100.0	69.9
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	69.9
16. UNIT CAPACITY FACTOR (USING MDC) (%)	102.9	102.9	64.7
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	98.9	98.9	62.2
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	12.4

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
No Outages Scheduled

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373  
UNIT LASALLE TWO  
DATE February 10, 1993  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: January 1993

DAY	POWER	DAY	POWER
1	1,017	17	1,083
2	1,038	18	1,094
3	982	19	1,091
4	1,012	20	1,097
5	1,014	21	1,070
6	1,012	22	1,077
7	1,097	23	1,097
8	1,071	24	1,022
9	1,089	25	1,094
10	1,037	26	1,053
11	1,096	27	1,074
12	1,059	28	1,094
13	1,058	29	1,089
14	1,098	30	1,092
15	1,072	31	1,068
16	1,083		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%  
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVE # if applicable)
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(None)

## SUMMARY OF OPERATION:

The unit remained on-line at high power throughout the month. Several minor power reductions were required due to system load and surveillance activities.

F. UNIQUE REPORTING REQUIREMENTS (UNIT 2)

1. Safety Relief valve operations  
(None)
2. ECCS System Outages  
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual  
(None)
4. Major Changes to Radioactive Waste Treatment Systems  
(None)
5. Indications of Failed Fuel Elements  
(None)

## (UNIT 2)

Table 5

## F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

OUTAGE NO.	EQUIPMENT	PURPOSE
0005	2E21-C001	Lubrication
0011	2E51-F064 2E51-F091	Administrative control
0012	2DG01K	Instrument calibrations
0029	2DG01K	Lubrication
0034	2E22-M010	Synchroscope replacement
0038	2E22-S001	Lubrication
0040	2E22-S001	Spring charging motor replacement