

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Washington Nuclear Plant - Unit 2

DOCKET NUMBER (2)

05000897 1 OF 13

PAGE (3)

TITLE (4)

TIP Purge Line Design

EVENT DATE (8)			LER NUMBER (5)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (6)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)	
06	21	85	85	042	0	07	11	85			05000897	
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)												
OPERATING MODE (9)			20.402(b)			20.408(a)			90.73(a)(2)(iv)			73.71(b)
POWER LEVEL (10)			20.408(a)(1)(i)			90.36(a)(1)			90.73(a)(2)(v)			73.71(a)
0010			20.408(a)(1)(ii)			90.36(a)(2)			90.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 368A)
			20.408(a)(1)(iii)			90.73(a)(2)(ii)			90.73(a)(2)(vii)(A)			
			20.408(a)(1)(iv)			90.73(a)(2)(iii)			90.73(a)(2)(viii)(B)			
			20.408(a)(1)(v)			90.73(a)(2)(iv)			90.73(a)(2)(ix)			

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
AREA CODE	
R. L. Koenigs, Compliance Engineer	5019 317171-1215 011

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext 2279

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 21, 1985, the primary containment isolation check valve in the Traversing Incore Probe (TIP) purge line was identified by design engineering to be in noncompliance with License Commitments. A redesign was immediately prepared and implementation was completed June 26, 1985.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Washington Nuclear Plant - Unit 2	05100039785	04	2	000	2	OF 03

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

- a) Power Level - 0%
- b) Reactor Mode - 4 (Cold Shutdown)

Event

On June 21, 1985, during reviews for the annual FSAR update, an error was discovered in the original design/construction of the Traversing Incore Probe (TIP) system nitrogen purge line. Design drawings specified the check valve (and tubing to primary containment) to be ANSI B31.1 (Code Group D) rather than ASME Section III, Class 2 (Code Group B). No evidence has been found that this configuration was documented and dispositioned during construction completion.

Immediate Corrective Action

The TIP line was disconnected, the penetration was provided with a blank flange meeting Code Group B requirements, and an alternate nitrogen purge supply was obtained from within the drywell. The blank flange was successfully subjected to a local leak rate test.

Further Corrective Action

Further investigation will be made of the TIP system purge requirements to assure the current configuration is the optimum design for WNP-2.

This was the only penetration identified on the flow diagram (M604) that had an improper Code Group specified. To ensure that no common-mode design deficiency exists, other penetrations of similar design will be evaluated for compliance with primary containment design requirements.

Safety Significance

This penetration must comply with 10CFR50, Appendix A, General Design Criterion 56 and Regulatory Guide 1.11, Rev. 0 requirements for an instrument line communicating with the primary containment atmosphere. The physical configuration requirements were met by the original design (a single check valve outside the primary containment), but the Supply System's commitment to meet Code Group B requirements for this valve and line were not met. Should the line and/or valve have failed outside the primary containment, the remaining penetration hardware would have limited flow to a volumetric flow rate well below the rated capacity of one standby gas treatment train. Technically, there is no expectation of failure of B31.1 stainless steel tubing and a 5000# check valve in this application, however, ASME III fabrication and installation would have provided greater assurance that components will not fail under accident conditions.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

Similar Events

Not Applicable

EIIS Information

Text Reference

EIIS Reference

System

Component

TIP Purge Check Valve

IG

ISV

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

July 11, 1985

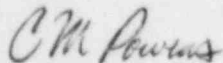
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-042

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-042 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,



C.M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:1a

Enclosure:
Licensee Event Report No. 85-042

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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