



Commonwealth Edison  
Braidwood Nuclear Power Station  
Route #1, Box 84  
Braceville, Illinois 60407  
Telephone 815/458-2801

February 10, 1993  
BW/93-0053

Director, Office of Resource Management  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering Braidwood Nuclear Power Station for the period January 1 through January 31, 1993.

K. L. Kofron  
Station Manager  
Braidwood Nuclear Station

KLK/JL/dla  
(731/ZD85G)

Attachments

cc: A. B. Davis, N.P.C., Region III  
J. Hickman, U.S.N.R.C.  
IL Dept. of Nuclear Safety  
N.R.C Resident Inspector, Braidwood  
INPO Records Center  
M. J. Wallace, V.P., Chief Nucl. Op. Officer  
D. L. Farrar, Nucl. Reg. Serv. Mgr.  
T. W. Simpkin, Nucl. Licensing Admin.  
D. R. Eggett, Nucl. Eng. Dept.  
S. M. Berg, Site V.P., Braidwood  
Nucl. Fuel Serv., PWR Plant Support  
P-f. Monitoring Grp., Tech Staff, Braidwood  
Nuclear Group, Tech Staff Braidwood

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BRAIDWOOD NUCLEAR POWER STATION

UNIT 1 AND UNIT 2

MONTHLY PERFORMANCE REPORT

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-456, LICENSE NO. NPF-72

NRC DOCKET NO. 050-457, LICENSE NO. NPF-77

I. Monthly Report for Braidwood Unit 1

A. Summary of Operating Experience

Braidwood Unit 1 entered the month of January, 1993 at approximately 100% capable. On January 7, the Unit tripped due to a failed logic card in the Solid State Protection System. The logic card was replaced. Unrelated corrective maintenance was also performed. The Unit returned to service on January 14, and operated routinely through the end of the month.

B. OPERATING DATA REPORT

DOCKET NO.: 50-456  
UNIT: Braidwood 1  
DATE: 02/10/93  
COMPILED BY: P. Stanczak  
TELEPHONE: (815)458-2801  
Ext. 2486

## OPERATING STATUS

1. Reporting Period: January, 1993 Gross Hours: 744

2. Currently Authorized Power Level (MWt): 3411  
Design Electrical Rating (MWe-gross): 1175  
Design Electrical Rating (MWe-net): 1120  
Max Dependable Capacity (MWe-gross): 1175  
Max Dependable Capacity (MWe-net): 1120

3. Power level to which restricted (If Any): None

4. Reasons for restriction (If Any): None

	THIS MONTH	YR TO DATE	CUMULATIVE
5. Report Period Hours:	744.0	744.0	39537.0
6. Hours Reactor Critical	584.3	584.3	30101.5
7. RX Reserve Shutdown Hours:	0.0	0.0	0.0
8. Hours Generator on Line	577.5	577.5	29553.2
9. Unit Reserve Shutdown Hours:	0.0	0.0	0.0
10. Gross Thermal Energy (MWh)	1810443.0	1810443.0	88712083.0
11. Gross Elect. Energy (MWh)	632108.0	632108.0	30420756.0
12. Net Elect. Energy (MWh):	606092.0	606092.0	29055787.0
13. Reactor Service Factor:	78.5	78.5	76.1
14. Reactor Availability Factor:	78.5	78.5	76.1
15. Unit Service Factor:	77.6	77.6	74.7
16. Unit Availability Factor:	77.6	77.6	74.7
17. Unit Capacity Factor (MDC NET):	72.7	72.7	65.6
18. Unit Capacity Factor (DER NET):	72.7	72.7	65.6
19. Unit Forced Outage Rate:	22.4	22.4	10.3
20. Unit Forced Outage Hours:	166.5	166.5	3402.8
21. Shutdowns Scheduled Over Next 6 months:	N/A		
22. If Shutdown at End of Report Period, Estimated Date of Start-up:	N/A		

## C. AVERAGE DAILY UNIT NET POWER LEVEL LOG

DOCKET NO.: 50-456  
UNIT: Braidwood 1  
DATE: 02/10/93  
COMPILED BY: P. Stanczak  
TELEPHONE: (815)458-2801  
Ext. 2486

MONTH: January, 1993

1. 1002	17. 1109
2. 1072	18. 1114
3. 1011	19. 1120
4. 1064	20. 1115
5. 1049	21. 1095
6. 1036	22. * 1122
7. 623	23. 1064
8. - 14	24. 979
9. - 14	25. * 1122
10. - 14	26. * 1128
11. - 14	27. * 1146
12. - 15	28. * 1146
13. - 15	29. * 1144
14. 89	30. * 1147
15. 669	31. * 1139
16. 1021	

## INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

\* Due to condenser efficiency.

D. UNIT SHUTDOWNS/REDUCTIONS

DOCKET NO.: 50-456  
UNIT: Braidwood 1  
DATE: 02/10/93  
COMPILED BY: P. Stanczak  
TELEPHONE: (815)458-2801  
Ext. 2486

REPORT PERIOD: January, 1993

No	DATE	TYPE	HOURS	REASON	METHOD	LER NUMBER	SYSTEM	COMPONENT	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
9	930107	F	166.5	4	3	93-001	EF	ECBD	A reactor trip occurred, due to a failed logic card, during testing of the reactor coolant pump under-frequency logic trip circuitry. The logic card was replaced. Other unrelated activities while shutdown included replacement of a leaking pressurizer safety valve and a chemistry test.

TYPE	REASON	METHOD	SYSTEM & COMPONENT
F-Forced	A-Equipment Failure Maint or Test	1 - Method	Exhibit F & H
S-Scheduled	B-Maint or Test	2 - Manual Scram	Instructions for Preparation of
	C-Refueling	3 - Auto Scram	Data Entry Sheet
	D-Regulatory Restriction	4 - Continued	Licensee Event Report
	E-Operator Training & License Examination	5 - Reduced Load	(LER) File (NUREG-0161)
	F-Administration	9 - Other	
	G-Oper Error		
	H-Other		



E. UNIQUE REPORTING REQUIREMENTS - UNIT 1

1. Safety/Relief valve operations.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT
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None

2. Licensee generated changes to ODCM.

None

F. LICENSEE EVENT REPORTS - UNIT 1

The following is a tabular summary of all Licensee Event Reports submitted during the reporting period, January 1 through January 31, 1993. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10 CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Report Date</u>	<u>Title of Occurrence</u>
93-001	01/29/93	Reactor Trip Due to Logic Card Failure in Train B Solid State Protection System



## II. Monthly Report for Braidwood Unit 2

### A. Summary of Operating Experience

Braidwood Unit 2 entered the month of January, 1993 at approximately 98% capable. On January 16, load was reduced to add oil to the 2C reactor coolant pump motor, and to allow tube leak repairs on the condenser. The Unit operated routinely through the end of the month.

B. OPERATING DATA REPORT

DOCKET NO.: 50-457  
UNIT: Braidwood 2  
DATE: 02/10/93  
COMPILED BY: P. Stanczak  
TELEPHONE: (815)458-2801  
Ext. 2485

## OPERATING STATUS

1. Reporting Period: January, 1993 Gross Hours: 744

2. Currently Authorized Power Level (MWt): 3411  
Design Electrical Rating (MWe-gross): 1175  
Design Electrical Rating (MWe-net): 1120  
Max Dependable Capacity (MWe-gross): 1175  
Max Dependable Capacity (MWe-net): 1120

3. Power level to which restricted (If Any): None

4. Reasons for restriction (If Any): None

	THIS MONTH	YR TO DATE	CUMULATIVE
5. Report Period Hours:	744.0	744.0	37619.0
6. Hours Reactor Critical	744.0	744.0	31906.5
7. RX Reserve Shutdown Hours:	0.0	0.0	0.0
8. Hours Generator on Line	744.0	744.0	31631.8
9. Unit Reserve Shutdown Hours:	0.0	0.0	0.0
10. Gross Thermal Energy (MWhT)	2399465.0	2399465.0	94412208.0
11. Gross Elect. Energy (MWh)	809473.0	809473.0	32266194.0
12. Net Elect. Energy (MWh):	777753.0	777753.0	30891789.0
13. Reactor Service Factor:	100.0	100.0	84.8
14. Reactor Availability Factor:	100.0	100.0	84.8
15. Unit Service Factor:	100.0	100.0	84.1
16. Unit Availability Factor:	100.0	100.0	84.1
17. Unit Capacity Factor (MDC NET):	93.3	93.3	73.3
18. Unit Capacity Factor (DER NET):	93.3	93.3	73.3
19. Unit Forced Outage Rate:	0.0	0.0	3.6
20. Unit Forced Outage Hours:	0.0	0.0	1197.3
21. Shutdowns Scheduled Over Next 6 Months:	3rd Refuel Outage, March, 1993		
22. If Shutdown at End of Report Period, Estimated Date of Start-up:	N/A		

C. AVERAGE DAILY UNIT NET POWER LEVEL LOG

DOCKET NO.: 50-457  
UNIT: Braidwood 2  
DATE: 02/10/93  
COMPILED BY: P. Stanczak  
TELEPHONE: (815)458-2801  
Ext. 2486

MONTH: January, 1993

1. _____	1080	17. _____	458
2. _____	1057	18. _____	908
3. _____	1012	19. _____	1101
4. _____	1060	20. _____	1102
5. _____	1093	21. _____	1096
6. _____	1073	22. _____	1061
7. _____	1054	23. _____	1103
8. _____	1091	24. _____	1040
9. _____	1101	25. _____	1105
10. _____	1093	26. _____	1105
11. _____	1033	27. _____	1106
12. _____	1084	28. _____	1100
13. _____	1084	29. _____	1076
14. _____	1093	30. _____	1099
15. _____	1091	31. _____	1009
16. _____	824		

## INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

D. UNIT SHUTDOWNS/REDUCTIONS

DOCKET NO.: 50-457  
 UNIT: Braidwood 2  
 DATE: 02/10/93  
 COMPILED BY: P. Stanczak  
 TELEPHONE: (815)458-2801  
 Ext. 2486

REPORT PERIOD: **January, 1993**

No	DATE	TYPE	HOURS	REASON	METHOD	LER NUMBER	SYSTEM	COMPONENT	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
19	930116	S	0	B	5	N/A	AB	MO	Load was reduced to allow personnel to enter containment to add oil to the 2C reactor coolant pump motor, and to allow tube leak repairs on the main condenser.

TYPE	REASON	METHOD	SYSTEM & COMPONENT
F-Forced	A-Equipment Failure Maint or Test	1 - Method	Exhibit F & H
S-Scheduled	B-Maint or Test	2 - Manual Scram	Instructions for Preparation of
	C-Refueling	3 - Auto Scram	Data Entry Sheet
	D-Regulatory Restriction	4 - Continued	Licensee Event Report
	E-Operator Training & License Examination	5 - Reduced Load	(LER) File (NUREG-0151)
	F-Administration	9 - Other	
	G-Oper Error		
(731/ZD85G)13	H-Other		

E. UNIQUE REPORTING REQUIREMENTS - UNIT 2

1. Safety/Relief valve operations.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT
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None

2. Licensee generated changes to ODCM.

None

F. LICENSEE EVENT REPORTS - UNIT 2

The following is a tabular summary of all Licensee Event Reports submitted during the reporting period, January 1 through January 31, 1993. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10 CFR 50.73.

<u>Licensee Event</u> <u>Report Number</u>	<u>Report</u> <u>Date</u>	<u>Title of Occurrence</u>
None		