

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT II										DOCKET NUMBER (2) 0 5 0 0 0 3 6 6										PAGE (3) 1 OF 2			
TITLE (4) UNPLANNED REACTOR SCRAM DUE TO SUSPECTED MSR HIGH LEVEL																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)								
0	6	0	3	8	5	8	5	0	1	8	0	0	0	7	0	1	8	5	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																					
1		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)									
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)									
0 9 0		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME												TELEPHONE NUMBER											
Steven B. Tipps, Superintendent of Regulatory Compliance												9 1 2 3 6 7 1 7 8 5 1											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs													
X	L/S	S/N		R/3/6/9	N																		
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR					
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 06/03/85 at approximately 2104 CDT, with the Unit operating at 2187 MWt (i.e., approximately 90% power), during a load reduction to take a division of Plant Service Water (PSW) out of service, a Turbine Trip and subsequent Reactor Scram was received on a high Moisture Separator Reheater (MSR) level signal.

The transient proceeded smoothly. Reactor water level decreased to + 2 inches (reference instrument zero), which is 146.5 inches above the top of the fuel. A group 2 primary containment isolation signal was received and the valves closed as required. Reactor water level was quickly recovered via the "A" feedwater pump. The turbine bypass valves opened to reduce reactor pressure, which reached 1080 psig.

An investigation of the suspected MSR high level revealed that the most likely cause of this event was due to a spurious high level trip signal from the "D" MSR level switch (2N38-N308D). The level switch was inspected for failure; however, no evident failure was found.

No actual or potential safety consequences resulted from this event. The health and safety of the public were not affected by this event. There are no known previous similar events.

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PDR ADOCK 05000366
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
EDWIN I. HATCH, UNIT II	0 5 0 0 0 3 6 6 8 5	—	0 1 8	—	0 0 0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

This 30 day LER is required by 10CFR50.73(a)(2)(iv) due to an unplanned Reactor Scram.

On 06/03/85 at approximately 2104 CDT, with the unit operating at 2187 MWt (i.e., approximately 90% power), during a load reduction to take a division of Plant Service Water (PSW) out of service, a Turbine Trip and subsequent Reactor Scram was received on a high Moisture Separator Reheater (MSR) level signal.

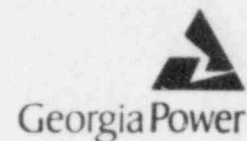
The transient proceeded smoothly. Reactor water level decreased to + 2 inches (reference instrument zero), which is 146.5 inches above the top of the fuel. A group 2 primary containment isolation signal was received and the valves closed as required. Reactor water level was quickly recovered via the "A" feedwater pump. HPCI and RCIC remained operable and in standby. However, neither HPCI nor RCIC was needed to control level. The turbine bypass valves opened to reduce reactor pressure, which reached 1080 psig. The SRV's were not required to operate because reactor pressure did not reach the SRV's setpoint.

An investigation of the suspected MSR high level revealed that the most likely cause of this event was due to a spurious high level trip signal from the "D" MSR level switch (2N38-N308D). The level switch was inspected for failure; however, no evident failure was found.

A Design Change Request (DCR) was initiated to change the MSR level trip logic. The existing trip logic only requires actuation of 1 level switch for the trip function. However, the DCR requires a logic change such that 2 of 3 level switches actuate for the trip function.

No actual or potential safety consequences resulted from this event. The health and safety of the public were not affected by this event. There are no known previous similar events.

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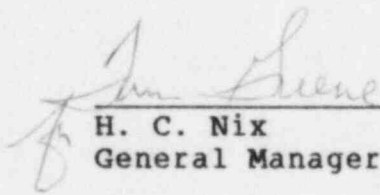
Edwin I. Hatch Nuclear Plant

July 1, 1985
GM-85-618

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-018. This report is required by 10CFR 50.73(a)(2)(iv).



H. C. Nix
General Manager

HCN/SBT/vlz

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