



**BOSTON EDISON**

Pilgrim Nuclear Power Station  
Rocky Hill Road  
Plymouth, Massachusetts 02360

**E. S. Kraft, Jr.**

Vice President Nuclear Operations  
and Station Director

February 11, 1993  
BECO Ltr. #93-015

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

License No. DPR-35  
Docket No. 50-293

January 1993 Monthly Report

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

*E. S. Kraft, Jr.*  
E. S. Kraft, Jr.

WJM/bal

Attachment

cc: Mr. Thomas T. Martin  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
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# OPERATING DATA REPORT

DOCKET NO. 50-293  
 DATE February 11, 1993  
 COMPLETED BY: W. Munro  
 TELEPHONE (508) 747-8474

## OPERATING STATUS

## NOTES

1. Unit Name Pilgrim I
2. Reporting Period January 1993
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. if Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:  
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	744.0	744.0	176616.0
12. Number of Hours Reactor Was Critical	744.0	744.0	106602.6
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	744.0	744.0	102650.4
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated(MWH)	1479408.0	1479408.0	179488224.0
17. Gross Electrical Energy Generated (MWH)	512540.0	512540.0	60646534.0
18. Net Electrical Energy Generated (MWH)	493387.0	493387.0	58286303.0
19. Unit Service Factor	100.0	100.0	58.1
20. Unit Availability Factor	100.0	100.0	58.1
21. Unit Capacity Factor (Using MDC Net)	99.0	99.0	49.3
22. Unit Capacity Factor (Using DER Net)	101.2	101.2	50.4
23. Unit Forced Outage Rate	0.0	0.0	12.2
24. Shutdown Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling Outage #9 starting 4-3-93 for a duration of 58 days.			

25. If Shut Down At End of Report Period, Estimated Date of Startup Unit Operating.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-292  
UNIT Pilgrim I  
DATE February 11, 1993  
COMPLETED BY: W. Munro  
TELEPHONE 508) 747-8474

MONTH January 1993

DAY	AVERAGE DAILY POWER LEVEL (MWe Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	667	16	638
2	668	17	669
3	666	18	669
4	665	19	669
5	665	20	669
6	663	21	668
7	664	22	668
8	662	23	666
9	664	24	667
10	665	25	667
11	665	26	666
12	664	27	666
13	665	28	667
14	664	29	666
15	606	30	666
		31	665

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY  
PILGRIM NUCLEAR POWER STATION  
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR JANUARY 1993

The unit started the reporting period at approximately 100 percent core thermal power (CTP), and was essentially maintained at that level until 1-15-93 when power was reduced to approximately 67 percent CTP to perform a main condenser backwash. On 1-16-93 the unit attained 100 percent CTP where it was maintained for the remainder of the reporting period. Minor power reductions were made on 1-2, 1-7, 1-14, 1-16 and 1-23-93 to perform control rod exercises.

SAFETY RELIEF VALVE CHALLENGES  
Month of January 1993

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operator via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 date 01/05/81.

## REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: April 3, 1993
3. Scheduled date for restart following next refueling: May 30, 1993
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1991 refueling outage was of the same design as loaded in the previous outage and consisted of 168 assemblies.
7.
  - (a) There are 580 fuel assemblies in the core.
  - (b) There are 1489 fuel assemblies in the spent fuel pool.
8.
  - (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
  - (b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 831 fuel assemblies.

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Diesel Generators and Auxiliaries	Emergency Diesel Generator 'A' Air Compressor Battery	Cell #4 found with low level. (PR 93.9002)	Crack in outer battery casing due to aging.	Replaced battery	Continue to perform monthly surveillance on battery.	N/A
High Pressure Coolant Injection (HPCI) System	HPCI Flow Indicating Controller FIC 2340-1	No indication of flow while performing HPCI operability surveillance due to blown power supply fuse inside FIC 2340-1 (PR 93.9021)	Root cause under investigation.	Replaced blown fuse and performed surveillance satisfactorily	Refer to associated LER	LER 93-001-00 (to be issued)
Salt Service Water (SSW) System	SSW Pump P-208G	High vibration observed while performing SSW Pump and Valve Surveillance Procedure 8.5.3.2.	Pump degradation	Balanced pump motor coupling	Investigating new pump design (Spec. M8-B). LTP #617 SSW Water Pump Upgrade Program.	N/A
Salt Service Water (SSW) System	Motor Operator MO-3801, TBCCW Heat Exchanger "A" Service Water Outlet Valve.	Failed to operate via the control switch on Panel C1 in the Control Room.	Root Cause under investigation	Overhauled motor operator and HBC gear unit. Replaced spring pack and motor pinion. Replaced both bearings in the HBC worm gear. Installed new SMB and HBC gasket kits.	To be determined	N/A

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

MONTH January, 1993

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Reactor Building Closed Cooling Water (RBCCW) System	RBCCW Loop "B" Inlet Valve Motor Operator MO-4010B.	Failed to operate from the control switch, and position indication was erratic.	Rcot Cause under investigation	Adjustment of spring tension on limit switches 4 & 8. Following adjustment the valve was operated satisfactorily.	To be determined	N/A

## UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293

DOCKET NO: 50-293  
 NAME: Pilgrim I  
 DATE: February 11, 1993  
 COMPLETED BY: W. Munro  
 TELEPHONE: 508) 747-8474  
 REPORT MONTH January 1993

NO.	DATE	TYPE1	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE4	COMPONENT CODE5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
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There were no unit shutdowns or reportable power reductions during the reporting period.

1	2	2	3	4&5
F-FORCED S-SCHED	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other(LER) File (NUREG-1022)	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report