

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT II										DOCKET NUMBER (2) 0 5 0 0 0 3 5 6 1 OF 0 2										PAGE (3) 1 OF 0 2																					
TITLE (4) ESF ACTUATION																																									
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																							
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)								
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0 4			2 0			8 5			8 5			0 0			8			0 0			0 5			2 0			8 5									0 5 0 0 0					
OPERATING MODE (9) 5						THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																																			
POWER LEVEL (10) 0 0 1 0						20.402(b)						20.406(c)						<input checked="" type="checkbox"/> 50.73(a)(2)(iv)						73.71(b)																	
						20.406(a)(1)(i)						50.36(c)(1)						50.73(a)(2)(v)						73.71(e)																	
						20.406(a)(1)(ii)						50.36(c)(2)						50.73(a)(2)(vii)						OTHER (Specify in Abstract below and in Text, NRC Form 366A)																	
						20.406(a)(1)(iii)						50.73(a)(2)(i)						50.73(a)(2)(viii)(A)																							
						20.406(a)(1)(iv)						50.73(a)(2)(ii)						50.73(a)(2)(viii)(B)																							
20.406(a)(1)(v)						50.73(a)(2)(iii)						50.73(a)(2)(ix)																													
LICENSEE CONTACT FOR THIS LER (12)																																									
NAME Steven B. Tipps, Superintendent of Regulatory Compliance														TELEPHONE NUMBER 9 1 1 2 3 6 1 7 + 1 7 8 1 5 1 1																											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																									
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPROS				CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NPROS																					
N/A																																									
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR																					
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)														<input checked="" type="checkbox"/> NO																											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 04/20/85, at approximately 1054 CST, with the reactor mode switch in the refuel position and reactor power at 0 MWt, the Reactor Water Cleanup outboard isolation valve (2G31-F004) isolated on high differential flow.

The high differential flow, and the isolation of 2G31-F004 occurred when plant personnel were valving in (i.e., returning to service after the completion of backwash and precoat) the "2A" RWCU filter demineralizer. The high differential flow is believed to be the result of flow fluctuation during the valving in of the "2A" filter demineralizer.

The 2G31-F004 was opened at approximately 1058 CST. "2A" RWCU filter demineralizer was returned to service at approximately 1235 CST on 04/20/85.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT II	0 5 0 0 0 3 6 6 8 5	—	0 0 8	—	0 0 0	2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR 50.73(a)(2)(iv) since the Reactor Water Clean-up (RWCU) Outboard Isolation Valve (2G31-F004) is a Primary Containment Isolation Valve and its isolation constitutes the actuation of an Engineered Safety Feature (ESF).

On 04/20/85, at approximately 1054 CST, with the reactor mode switch in the refuel position and reactor power at 0 MWT, the Reactor Water Cleanup outboard isolation valve (2G31-F004) isolated on high differential flow.

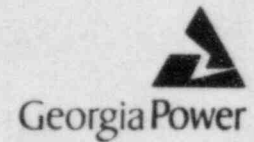
The high differential flow, and the isolation of 2G31-F004 occurred when plant personnel were valving in (i.e., returning to service after the completion of backwash and precoat) the "2A" RWCU filter demineralizer. The high differential flow is believed to be the result of flow fluctuation during the valving in of the "2A" filter demineralizer. This was not the result of procedural or personnel errors. No corrective action is required.

The 2G31-F004 was opened at approximately 1058 CST. "2A" RWCU filter demineralizer was returned to service at approximately 1235 CST on 04/20/85.

No other isolation of RWCU due to this same cause has occurred on Unit 2. A similar isolation of RWCU on Unit 1 has occurred due to differential flow during the process of returning a filter demineralizer to service (refer to LER 50-321/1985-012); this event was caused by plant personnel having to position manual valves in the RWCU room, and then exiting the room to control the flow.

Similar events (i.e., isolation of RWCU primary containment isolation valves for other causes) were reported via LERs 50-321/1984-029, 50-321/1985-001, and 50-366/1984-007, 010, 012, 019, and 024 and 50-366/1985-003, and 007, Rev. 1.

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Edwin I. Hatch Nuclear Plant

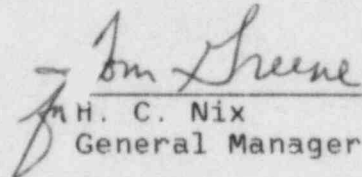
May 20, 1985

GM-85-474

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-008. This report is required by 10CFR 50.73(a)(2)(iv).


J. H. C. Nix
General Manager

SB1
HCN/GBT/vlz

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