Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

February 12, 1993
LIC-93-0075

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

REFERENCE: Docket No. 50-285

Gentlemen:

SUBJECT: January 1993 Monthly Operating Report (MOR)

Enclosed is the January 1993 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification Section 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/sel

Enclosure

c: LeBoeuf, Lamb, Leiby & MacRae
J. L. Milhoan, NRC Regional Administrator, Region IV
R. P. Mullikin, NRC Senior Resident Inspector
S. D. Bloom, NRC Project Manager
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
NRC Office of Management & Program Analysis (2)
INPO Records Center
American Nuclear Insurers

IE24

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

January 1993
Monthly Operating Report

1. OPERATIONS SUMMARY

During January 1993, Fort Calhoun Station operated at 100% power. An operational restriction on control element assembly (CEA) movement which was in place since October 1992 as a result of the exchange of two control channels with two safety channels of nuclear instrumentation has been lifted. The analyses to support operation in a rodded configuration have been completed which allowed lifting of the restriction.

In January 1993, modification work on the switchgear halon system continued and a component cooling water (CCW) heat exchanger was cleaned.

A Technical Specification (TS) surveillance issue relative to boric acid storage tank (BAST) level instrumentation was discovered during the month. Technical Specifications require a daily check of BAST level via two independent level sensors. However, it was discovered that the two sources previously used (local & remote level indication) were from the same level indicator. A temporary modification installed Tygon tubing on each tank to provide alternate level indication. The temporary modification, in conjunction with the existing instruments, returned the system to compliance. Details are contained in LER 93-001 dated February 8, 1993. A modification to permanently correct the problem is planned.

The following NRC inspections were completed during January 1993:

IER No.	Title
92-33	Routine Resident Inspection
93-01	Self-Assessment Activities
93-02	Routine Resident Inspection

The following LERs were submitted during this reporting period:

LER No.	Description
92-031	Inoperability of Fire Water Suppression System
92-032	Failure to Satisfy Fire Watch Requirements for Impaired Halon System

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

There were no safety valve or PORV challenges or failures during January 1993.

3. RESULTS OF LEAK RATE TESTS

The reactor coolant system (RCS) leak rate averaged less than 0.250 gpm for the month of January. On January 27, "known" leakage to the reactor coolant drain tank (RCDT) increased to 0.396 gpm before decreasing several hours later to approximately 0.168 gpm. With the exception of this single fluctuation, the leak rate remained relatively steady with no degrading trends noted.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
149	This amendment makes changes to Technical Specifications 5.5 and 5.8 to reflect the implementation of a Qualified Reviewer Program for the review and approval of new procedures, and changes thereto, at Fort Calhoun Station.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF JANUARY 1993

Cleaned and sandblasted CCW Heat Exchanger AC-1. Applied an epoxy coating to the tube sheets.

Rebuilt actuators and replaced springs on CCW Inlet Valves HCV-491A & HCV-491B for Component Cooling Heat Exchanger AC-1C per Engineering Change Notice (ECN 91-450).

Replaced a broken spring on the actuator for Suction Valve HCV-2937 for Low Pressure Safety Injection Pump SI-1B.

Repaired a pin-hole leak on the CCW Flange to the Oil Cooler and replaced the packing on Charging Pump CH-1C.

Replaced the packing on Spent Fuel Pool Cooling Pump AC-5B

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

Attachment I

OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE FEBRUARY 08 1993
COMPLETED BY M. L. EDWARDS
TELEPHONE (402) 636-2451

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: JANUARY 1993

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	744.0	169658.0
12. Number of Hours Reactor was Critical	744.0	744.0	131354.3
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	744.0	129807.2
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1112001.8	1112001.8	170677567.8
17. Gross Elec. Energy Generated (MWH)..	378130.0	378130.0	56218930.2
18. Net Elec. Energy Generated (MWH)....	361238.7	361238.7	53632125.6
19. Unit Service Factor.....	100.0	100.0	76.5
20. Unit Availability Factor.....	100.0	100.0	76.5
21. Unit Capacity Factor (using MDC Net)	101.6	101.6	68.7
22. Unit Capacity Factor (using DER Net)	101.6	101.6	66.9
23. Unit Forced Outage Rate.....	.0	.0	4.3

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
ON MAY 2, 1993, THE PLANT IS SCHEDULED TO BE PLACED IN HOT-SHUTDOWN FOR AN 8 DAY MID-CYCLE MAINTENANCE OUTAGE.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

Attachment II

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	FEBRUARY 08 1993
COMPLETED BY	M. L. EDWARDS
TELEPHONE	(402) 636-2451

MONTH JANUARY 1993

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	486	17	485
2	486	18	485
3	486	19	485
4	486	20	485
5	486	21	485
6	486	22	485
7	486	23	485
8	486	24	485
9	485	25	485
10	486	26	486
11	485	27	485
12	486	28	485
13	486	29	485
14	486	30	485
15	486	31	485
16	486		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE February 8, 1993
COMPLETED BY M. L. Edwards
TELEPHONE (402) 636-2451

REPORT MONTH January 1993

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
None	--	--	--	--	--	--	--	--	During January 1993, the plant operated at a nominal 100% power.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
4
Method:Exhibit G - Instructions
1-Manualfor Preparation of Data
2-Manual Scram.Entry Sheets for Licensee
3-Automatic Scram.Event Report (LER) File (NUREG-0161)
4-Other (Explain)

5
Exhibit 1 - Same Source

(9/77)

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending January 1993

1. Scheduled dated for next refueling shutdown. — September 1993

2. Scheduled date for restart following refueling. — November 1993

3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Incorporate specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — June 1993

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — None Planned

6. The number of fuel assemblies:

a) in the core	133 Assemblies
b) in the spent fuel pool	529 Assemblies
c) spent fuel pool storage capacity	729 Assemblies
d) planned spent fuel pool storage capacity	Planned to be increased with high density spent fuel racks.

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1995*

* Capability of full core offload of 133 assemblies lost. Reracking to be performed between the 1993 and 1995 Refueling Outages.

Prepared by [signature] Date 2-9-93