QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JULY 1985

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Becky Brown and Dave Kimler, telephone number 309-654-2241, extensions 127 and 192.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

June 1-16: Unit One began the month of June shutdown from a Reactor scram on May 30. On June 1, at 1335 hours, the Reactor went critical, and was placed on line at 2152 hours. Full power was reached after a normal load increase on June 4 at 0500 hours. On June 5, at 0310 hours, load was dropped to 700 MWe for Turbine surveillances. At 0450 hours load began an increase to 800 MWe, and at 0550 hours the unit was placed on Economic Generation Control (EGC).

June 17-22: On June 17, at 2340 hours, a normal orderly shutdown commenced at 25 MWe/hour, because the 1/2 Diesel Generator was out of service and the Unit One RHR Service Water Pumps were inoperable concurrently, due to Technical Specification 3.9.E.1. On June 18, at 0325 hours, the shutdown was terminated due to the operability of the 1/2 Diesel Generator and the Unit One RHR Service Water Pumps, and a normal load increase to maximum power began. Maximum load was reached at 1400 hours.

June 23-30: On June 23, at 0315 hours, load began a drop to minimum power as per the Load Dispatcher. Minimum power was reached at 0620 hours. At 0925 hours load began an increase to full power, and the unit was placed on EGC, on June 24 at 2300 hours. On June 30, at 0300 hours, load was dropped to 600 MWe for Turbine weekly surveillances. At 0430 hours, load was decreased to 550 MWe for a Control Rod pattern adjustment. At 0515 hours, the unit began a normal load increase to full power.

B. Unit Two

June 1-13: Unit Two began the month shutdown for the End of Cycle Seven Refueling and Maintenance Outage. On June 5, at 2301 hours, the Reactor went critical and remained in HOT STANDBY until June 7, at 0214 hours, when the unit was placed on line. At 0538 hours the unit was taken off line for a Turbine Overspeed Test, and was placed back on line at 0628 hours. Maximum power was reached on June 13 at 0130 hours after a normal load increase.

June 14-22: On June 14, at 2200 hours, load began a drop to HOT STANDBY to repair an EHC oil leak. The unit went off line on June 15, at 0313 hours, and was returned on June 16, at 0052 hours. At 0120 hours, load began a normal increase to full power. On June 18 full power was reached at 2300 hours. On June 21, at 0900 hours, load was decreased to 750 MWe, and the unit was placed on EGC at 0955 hours.

II. SUMMARY OF OPERATING EXPERIENCE

(continued)

B. Unit Two (continued)

June 23-30: On June 23, at 0240 hours, load began a drop to minimum power as per Load Dispatcher. Minimum speed was reached at 0610 hours. At 0915 hours, load began a normal increase to full power. On June 24, at 2240 hours, the unit was holding load at 810 MWe.

On June 26, at 0310 hours, load began a decrease to 650 MWe as per Load Dispatcher. At 0425 hours load was holding at 650 MWe. At 0505 hours load was increased to maximum power. At 1845 hours load reached 812 MWe and held steady.

On June 28, at 0930 hours, load was decreased to 700 MWe to repair a leak in the vent of the 2B2 Feedwater Heater. At 2120 hours load began an increase to full power. On June 29, at 0915 hours, the unit reached 815 MWe and held load for the duration of June.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

In accordance with Technical Specification Section 6.7.B, a notification of changes to the Offsite Dose Calculation Manual (ODCM), Revision 11, was made. This notification was in the form of a letter dated April 26, 1985, to D. Eisenhut from Krista Licari, and was acknowledged by Avis M. Paulson on May 30, 1985.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q42744		Unit 1 Diesel Generator Flex Fuel Line From Day Tank	Fatigue and life of line (line was original part, 12 years old).	1/2 Diesel Generator was operable. All necessary surveillances were initiated.	Replaced fuel flex line and fittings.

UNIT 2 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q37513		Solenoid Valve on Unit 2 Diesel Generator Air Start System	1/2 Diesel Generator solenoid valve "bound up". Would not open on start signal.	1/2 Diesel Generator did not start. Alarm reset; the 1/2 Diesel Generator did start.	Solenoid valve replaced as preventative maintenance on all three Diesel Generators.
Q42127	85-006	AO 2-203-2B Outboard MSIV. Repaired Valve & Prepared for Second LLRT; Failed First LLRT.	Valve seating surface wear due to high pressure steam flow.	Leak rate exceeded Tech Spec 4.7.A.2.E., but minimum implications since Unit 2 was in SHUTDOWN mode.	Lapped main seat, replaced main disc, pilot seat, and pilot disc. Retested satisfactorily.
Q42193	85-007	MO 2-220-1 Main Steam Drain. Rebuilt & re-lapped, new grease. Failed LLRT.	Valve seating surface wear due to high pressure steam flow.	Leak rate exceeded Tech Spec limit 4.7.A.2.E., but minimal implications. Since Unit 2 was in COLD SHUTDOWN mode.	Cleaned valve internals and lapped the seat.
Q42508	85-013	Unit 2 Diesel Generator	Governor compensating mechanism was out of adjustment due to normal operational wear.	Unit 2 Diesel Generator tripped on overspeed while 1/2 Diesel Generator was out of service. Declared GSEP Unusual event, but was minimal since Unit 2 was in COLD SHUTDOWN.	The governor compensating mechanism was properly adjusted and successfully tested.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT ONE

Licensee Event Report Number	Date	Title of Occurrence
85-08	6-17-85	1C RHR Service Pump Inoperable; 1/2 Diesel Generator and 1B RHR Service Water Pump Inoperable

UNIT TWO

85-15	6-28-85	2A Reactor Building Ventilation Monitor Drift; Standby Gas Treatment System Auto-Start

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V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

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OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE JULY 8 1985

COMPLETED BY CAROL L KRONICH

TELEPHONE 309-654-2241x193

OPERATING STATUS

1. Reporting period: 0000 060185 2400 063085 Gross hours in reporting period: 720

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	706.4	4094.0	92416.4
6. Reactor reserve shutdown hours	0.0	0.0	3421.9
7. Hours generator on line	698.1	4036.4	89071.7
8. Unit reserve shutdown hours.	0.0	0.0	909.2
9. Gross thermal energy generated(MWH)	1633804	9481367	185227757
10. Gross electrical energy generated(MWH)	538583	3131841	59909292
11. Net electrical energy generated(MWH)	515297	2999088	55954790
12. Reactor service factor	98.1	94.3	80.2
13. Reactor availability factor	98.1	94.3	83.2
14. Unit service factor	97.0	92.9	77.3
15. Unit availability factor	97.0	92.9	78.1
16. Unit capacity factor (Using MDC)	93.1	89.8	63.2
17. Unit capacity factor (Using Des.MWe)	90.7	87.5	61.6
18. Unit forced outage rate	5.9	7.0	6.1

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE JULY 8 1985

COMPLETED BY CAROL L KRONICH

TELEPHONE 309-654-2241x193

OPERATING STATUS

1. Reporting period: 0000 060185 2400 063085 Gross hours in reporting period: 720

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	601.0	2231.8	87137.8
6. Reactor reserve shutdown hours	0.0	0.0	2985.8
7. Hours generator on line	551.3	2159.9	84209.1
8. Unit reserve shutdown hours.	0.0	0.0	702.9
9. Gross thermal energy generated(MWH)	1146225	4868847	176387914
10. Gross electrical energy generated(MWH)	369085	1586554	56239965
11. Net electrical energy generated(MWH)	352439	1514505	52832490
12. Reactor service factor	83.5	51.4	76.3
13. Reactor availability factor	83.5	51.4	78.9
14. Unit service factor	76.6	49.7	73.7
15. Unit availability factor	76.6	49.7	74.3
16. Unit capacity factor (Using MDC)	63.7	45.3	60.1
17. Unit capacity factor (Using Des.MWe)	62.0	44.2	58.6
18. Unit forced outage rate	0.0	8.1	8.4

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE JULY 8 1985

COMPLETED BY CAROL L KRONICH

TELEPHONE 309-654-2241x193

MONTH June 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	2.5	17.	776.2
2.	489.2	18.	778.9
3.	736.7	19.	771.6
4.	768.4	20.	737.8
5.	748.6	21.	721.1
6.	846.0	22.	751.0
7.	671.2	23.	622.6
8.	761.7	24.	771.7
9.	787.5	25.	739.4
10.	771.3	26.	752.2
11.	781.2	27.	781.2
12.	771.3	28.	741.1
13.	744.1	29.	765.6
14.	771.6	30.	630.5
15.	725.9		
16.	740.4		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265
UNIT TWO
DATE JULY 8 1985
COMPLETED BY CAROL L KRONICH
TELEPHONE 309-654-2241x193

MONTH June 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-5.2	17.	576.2
2.	-5.7	18.	725.3
3.	-5.8	19.	773.9
4.	-9.5	20.	778.4
5.	-10.4	21.	738.4
6.	-15.4	22.	717.1
7.	120.7	23.	615.1
8.	296.5	24.	745.3
9.	382.6	25.	771.1
10.	482.8	26.	735.1
11.	593.7	27.	774.2
12.	707.9	28.	713.8
13.	758.7	29.	763.8
14.	758.3	30.	748.0
15.	-6.8		
16.	392.2		

INSTRUCTIONS
On this form,list the average daily unit power level in MWe-Net for each day in the reporting month.Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit,there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line).In such cases,the average daily unit power output sheet should be footnoted to explain the apparent anomaly

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY C Kronich

DATE July 8, 1985

REPORT MONTH JUNE 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-30	850605	S	0.0	H	5		HA	TURBIN	Reduced load to 700 MWe for weekly Turbine surveillances
85-31	850617	F	0.0	D	5	85-08	ZZ	ZZZZZZ	Commenced orderly shutdown due to 1/2 Diesel Generator out of service and RHR Service Water pumps out of service concurrently (terminated shutdown - 3.75 hrs)
85-32	850623	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load per Load Dispatcher
85-33	850630	S	0.0	H	5		RC	CONROD	Reduced load to 550 MWe for Control Rod pattern adjustment

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit 2

COMPLETED BY C Kronich

DATE July 8, 1985

REPORT MONTH JUNE 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-21	850614	S	21.65	B	5		HB	XXXXXX	Reduced load and placed unit on HOT STANDBY to repair EHC oil leak
85-22	850621	S	0.0	H	5		XX	ZZZZZZ	Reduced load to 750 MWe to place unit on Economic Generation Control (EGC)
85-23	850623	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load per Load Dispatcher
85-24	850626	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load per Load Dispatcher
85-25	850628	F	0.0	B	5		HH	HTEXCH	Reduced load for 2B Feedwater Heater leak repair

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VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: One

Date: June 1, 1985

Valve Actuated	No. & Type of Actuation
1-203-3A	1 Manual

Plant Conditions: Reactor Pressure - 924 lbs

Description of Events: Surveillance Technical Specification 4.5.D.1.b

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with Reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS

PERFORMED ON UNIT 1 & 2 CONTROL

ROD DRIVES, FROM 1-1 TO 12-31-85

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				Max. Time For 90% Insertion	DESCRIPTION
		5	20	50	90		Technical Specification 3.3.C.1 &
		0.375	0.900	2.00	3.5	7 sec.	3.3.C.2 (Average Scram Insertion Time)
6-8	176	0.31	0.68	1.45	2.54	3.17 (F-7)	Unit 2 Hot Scram Timing (H-8 Out of Service but verified during cold timing)

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

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QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: Q1 Reload: 7 Cycle: 8

2. Scheduled date for next refueling shutdown: 1-2-86

3. Scheduled date for restart following refueling: 4-2-86

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   DECEMBER 19, 1985; IF LICENSING ACTION REQUIRED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE PLANNED AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2340

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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Q.C.O.S.R.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: Q2 Reload: 7 Cycle: 8

2. Scheduled date for next refueling shutdown: 10-6-86

3. Scheduled date for restart following refueling: 12-22-86

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   Yes. A routine MAPLHGR amendment has been approved as a preparatory change to allow a 10 CFR 50.59 review.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   Not Applicable.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   None planned at present time.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 176

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center

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Commonwealth Edison
Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

TKT-85-37

July 1, 1985

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555
Attention: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of June 1985.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

T. K. Tamlyn
Services Superintendent

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Enclosure

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