

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JUNE 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982, and commercial power operation was commenced on January 1, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

June 10-29

June 10, 1235 hours.	Reactor Critical.
June 11, 0100 hours.	Generator on-line.
June 11, 0700 hours.	Reactor power at 26.5%.
June 11, 2300 hours.	Reactor power at 70.9%.
June 13, 0100 hours.	Reactor power decreased to 58% in order to manipulate control rods.
June 13, 0700 hours.	Reactor power restored to 74.6%.
June 15, 0700 hours.	Reactor power at 84.7%.
June 17, 1915 hours.	Commenced normal Unit shutdown to repair Electro-Hydraulic control system.
June 18, 0500 hours.	Generator off-line.
June 18, 2300 hours.	Reactor power at 19.0%, Generator off-line.
June 19, 0605 hours.	Generator on-line.
June 19, 0807 hours.	Power reduction commenced due to loss of cooling to transformer 1E.
June 19, 0900 hours.	Power increased after cooling was restored to transformer 1E.
June 19, 1050 hours.	Turbine trip due to error in troubleshooting.
June 19, 1135 hours.	Turbine on-line.
June 21, 1500 hours.	Reactor power at 72%.
June 22, 0445 hours.	Reactor power decreased to 58.3% in order to manipulate control rods.
June 24, 0700 hours.	Reactor power restored to 86%.
June 24, 0745 hours.	Slight power reduction commenced due to oscillations in the number 3 Turbine Control Valve.
June 29, 0700 hours.	Reactor power at 91.5%.
June 29, 0815 hours.	Reactor scram on loss of CRD flow. The reactor had been critical for 451 hours and 40 minutes in June so far.

June 30

June 30, 0900 hours.	Reactor critical.
June 30, 2300 hours.	Reactor power at 6.3%; Generator off-line.

The reactor was critical for 466 hours and 40 minutes in June.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or Technical Specifications during this reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L36208	Division III 125 Volt Batteries cell number 28.	Faulty cell.	Potential for degraded voltage of Division III DC bus.	Replaced faulty cell.
L42234	Auxiliary Electrical Room HVAC Filter Train to charcoal filter. "A".	Jammed damper at inlet to charcoal filter.	Charcoal filter in continuous use.	Sent damper to vendor for repairs. Installed upon return.
L45143	Reactor Low Pressure RCIC Isolation Switches.	Leaky instrument stop valve.	No Significant Effects.	Replace Valve.
L46678	Auxiliary Electrical Equipment Room HVAC Filter train "A".	Damper at inlet to charcoal filter had a broken linkage.	Could not fully close damper.	Repaired Linkage.
L48201	Valve 1E12-F053A.	Failed Motor in operator.	Could not remotely operate valve; breaker kept tripping.	Replaced Motor.
L48319	Reactor Vessel Low-Low Level LPCS/RHR Initiation, ADS Permissive.	Five-Valve Manifold Leaked.	No Significant effects.	Replaced five-valve manifold.
L49353	Diesel Generator B High Water Temperature Switch.	Would not trip.	Diesel lost some protection from overheating.	Replaced switch.
L49358	Division II Battery Charger.	Charge Current Limiter out-of-tolerance.	Potential to overcharge Batteries.	Replaced printed circuit boards.
L49432	HCU Accumulator for CRD 02-27.	Leaky Valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L49440	"B" Narrow Range Level Indication.	Reference line partially empty.	Incorrect reading.	Back filled reference line.
L49472	HCU Accumulator for CRD 10-31.	Leaky Valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced Valve.
L49523	"A" Rod-Block Monitor.	Blown fuses.	Rod Block Monitor "A" did not read any local power range monitors.	Replaced fuses.
L49705	RCIC High Vessel Level Switch.	Out-of-Tolerance.	Signal to stop RCIC steam supply not generated when it should have been.	Recalibrated switch.
L49855	B RHR Water leg pump discharge control valve.	Worn Spring.	Would not fully close.	Replaced spring and gaskets.
L49905	"D" Inboard MSIV Leakage Control Flow Indication.	Flow indicator was dirty.	Incorrect Reading.	Cleaned indicator.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, June 1 through June 30, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-044-00	5-9-85	Chlorine Detector Actuation.
85-045-00	5-31-85	Manual Scram Due to Loss of Circulating Water.
85-046-00	6-17-85	Loss of both Diesel Fire Pumps.
85-047-00	6-10-85	Missed off-gas Hydrogen Sample.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE July 10, 1985
 COMPLETED BY Richard J. Rohrer
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: JUNE, 1985 GROSS HOURS IN REPORTING PERIOD: 720
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY): N/A
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|--|----------------|----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>466.7</u> | <u>3508.7</u> | <u>9790</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>102.9</u> | <u>1269</u> |
| 7. HOURS GENERATOR ON LINE | <u>405.3</u> | <u>3374.5</u> | <u>9431</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>1009176</u> | <u>9443063</u> | <u>26266352</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>321004</u> | <u>3109769</u> | <u>8580412</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>307031</u> | <u>2994945</u> | <u>8190007</u> |
| 12. REACTOR SERVICE FACTOR | <u>64.8%</u> | <u>80.3%</u> | <u>74.4%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>64.8%</u> | <u>82.7%</u> | <u>84.1%</u> |
| 14. UNIT SERVICE FACTOR | <u>56.3%</u> | <u>77.3%</u> | <u>71.7%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>56.3%</u> | <u>77.3%</u> | <u>71.7%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>41.2%</u> | <u>66.7%</u> | <u>60.1%</u> |
| 17. UNIT CAPACITY FACTOR(USING DESIGN MWe) | <u>39.6%</u> | <u>63.6%</u> | <u>57.8%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>43.7%</u> | <u>17.4%</u> | <u>15.5%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
 Unit one is scheduled for a refueling, maintenance, modification, and surveillance outage beginning September 3, 1985 and lasting 26 weeks.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: July 1, 1985

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373

UNIT: LASALLE ONE

DATE: July 10, 1985

COMPLETED BY: Richard J. Rohrer

TELEPHONE: (815) 357-6761

MONTH: JUNE, 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____ -6 _____	17. _____ 855 _____
2. _____ -7 _____	18. _____ 27 _____
3. _____ -5 _____	19. _____ 96 _____
4. _____ -6 _____	20. _____ 238 _____
5. _____ -8 _____	21. _____ 738 _____
6. _____ -9 _____	22. _____ 739 _____
7. _____ -7 _____	23. _____ 832 _____
8. _____ -6 _____	24. _____ 891 _____
9. _____ -7 _____	25. _____ 889 _____
10. _____ -14 _____	26. _____ 885 _____
11. _____ 357 _____	27. _____ 883 _____
12. _____ 775 _____	28. _____ 875 _____
13. _____ 797 _____	29. _____ 313 _____
14. _____ 902 _____	30. _____ -16 _____
15. _____ 897 _____	
16. _____ 895 _____	

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373

UNIT NAME LaSalle One

DATE JULY 10, 1985

COMPLETED BY Richard J. Rohrer

TELEPHONE (815)357-6761

REPORT MONTH JULY 1985

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
10	850531	F	241.00	A	2	Manually scrambled reactor following flooding of Lake Screen House.
11	850618	F	25.08	B	1	Took generator off-line to troubleshoot oscillations in Number 3 Turbine Control Valve.
12	850619	F	0.75	G	4	Turbine trip on high level signal during troubleshooting of "B" Level Switch.
13	850620	F	8.08	B	1	Took generator off-line to replace oil trip solenoid on turbine.
14	850629	F	39.75	A	3	Automatic scram caused by loss of CRD header flow when one CRD pump failed to remain "ON" after starting.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
6-1-85	1B21-F013M	1 Manual	170 psig	Manually opened
	1B21-F013N	1 Manual	110 psig	successively to provide
	1B21-F013F	1 Manual	85 psig	controlled cool-down of
	1B21-F013R	1 Manual	115 psig	the reactor following
	1B21-F013S	1 Manual	115 psig	Main Steam Line Isolation.

Valves 1B21-F013 A,B,C,D,E,F,G,H, and L were also actuated as part of this event. These actuations occurred on the evening of 5-31-85 and are reported in the Monthly Report for May, 1985.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-472-85	1B Diesel Generator	Surveillance.
1-479-85	1B Diesel Generator Cooling Water System.	Testing.
1-480-85	HPCS Diesel Generator Fuel Oil Transfer Pump.	Testing.
1-483-85	1E12-F099 A & B	Replace Handswitch.
1-489-85	"B" RHR Service Water Process Radiation Monitor.	Decontamination.
1-492-85	HPCS Suction Valve from cycled condensate and full flow test valve.	Isolate Leak.
1-505-85	Shutdown Cooling Suction Header.	Testing.
1-508-85	RHR Pump "A".	Sample oil.
1-512-85	HPCS Pump.	Sample and change oil.
1-517-85	"B" RHR Service Water Process Radiation Monitor.	Decontamination.
1-523-85	1E12-F332A	Throttle Valve.
1-539-85	B/C RHR Water Leg Pump.	Repair check valve.
1-571-85	"A" Diesel Generator and cooling water pump.	Change oil in pump.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation Manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

July 10, 1985

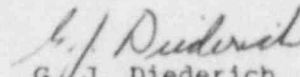
Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for the period June 1 through June 30,
1985.

Very truly yours,


G. J. Diederich
Station Manager
LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
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