



Commonwealth Edison

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RJW-93-03

February 8, 1993

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of January 1993.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

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RJW/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

January 1993

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One ended its refuel outage in December and began the month of January, 1993 at full power. The unit remained on line for the entire month and performed numerous load drops for the Chicago Load Dispatch (CLD). None of these load drops, however, caused the average daily unit power level to drop below 80% of full power.

B. Unit Two

Quad Cities Unit Two was transferring reactor protection system (RPS) power, at 10:08 on January 7, 1993, when an auto SCRAM was received. License event report (LER) number 265-93-001 explains that a faulty DC solenoid caused a main steam isolation valve (MSIV) to close suddenly. The resulting brief pressure transient caused the unit to SCRAM on high flux.

The unit was made critical and synchronized to the grid at 08:39 on January 13, 1993.

A second reactor SCRAM occurred, at 13:40 on January 29, 1993, from a unit 2 reactor high pressure signal. LER number 265-93-005 is currently being written to investigate the cause of this signal. It is suspected that maintenance work near the pressure instruments may have inadvertently caused an erroneous signal.

The unit was made critical and synchronized to the grid at 22:39 on January 31, 1993.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	WORK REQUESTED	WORK PERFORMED
Q02252	7300	REPAIR ACB ELECTRIC 480 VAC BREAKER.	FOUND THE "Y" RELAY WOULD NOT ENERGIZE. THE BREAKER CONTACTS WERE CLEANED AND A NEW CUTOFF SWITCH WAS INSTALLED.
Q03003	220	REPAIR PENCIL SIZE LEAK IN U1 B FEED WATER CHECK VALVE.	DISASSEMBLED VALVE AND REPLACED THE "O" RING.
Q03041	2500	U1 ACAD TO STANDBY GAS TREATMENT ISOLATION VALVE 1-2599-5B FAILED LLRT. REPAIR.	INSTALLED NEW SPRING IN OPERATOR. LLRT PASSED.
Q03047	2300	REPAIR BAD PACKING LEAK ON U1 HPCI STEAM SUPPLY TEST TAP VALVE 1-2301-16.	REPACKED VALVE WITH CHESTERTON PACKING.
Q03166	7800	REWORK BUTT SPLICE ON U1 MCC 18/19-5 CUBICLE E1 RECIRC PUMP SUCTION VALVE 202-4B.	INSTALLED AN IN LINE SPLICE BETWEEN THE STARTER AND CONTROL TRANSFORMER.
Q03200	5400	REPAIR SEVERE AIR LEAK ON SJAE SUCTION VALVE SO-1-5401A.	FOUND THE ASCO SOLENOID BLOWING BY. INSTALLED NEW ASCO VALVE AND VERIFIED PROPER OPERATION WITH NO LEAKS.
Q03268	1600	REPAIR/REPLACE AND ADJUST BROKEN LIMIT SWITCHES ON DW TO TORUS RING HEADER VACUUM BREAKER AO 1-1601-32A.	REPLACED BOTH SWITCHES.
Q03284	7800	INVESTIGATE/REPAIR MCC 18/19-5 CUBICLE A4, 1A REACTOR WATER MANIFOLD CROSSTIE VALVE.	FOUND MOTOR BARS DELAMINATED AND SEPARATED. REPLACED MOTOR WITH A NEW ONE.

Q03335	5700	INSTALL NEW SELECTOR SWITCH ON A RHR ROOM COOLER SELECTOR 1-5746A.	INSTALLED NEW SELECTOR SWITCH.
Q03554	6700	REPAIR BENT AND DISCOLORED CONTACT ARM ON RELAY 127-B131X3, BUS 13-1.	REPLACED RELAY.
Q03621	760	REMOVE AND REINSTALL IRM #14 DETECTOR.	INSTALLED NEW DETECTOR.
Q03654	1000	REPLACE VALVE STEM AND ACTUATOR ON RHR SERVICE WATER 1A HEAT EXCHANGER UPSTREAM STOP VALVE 1-1001-185A.	REPLACED ENTIRE VALVE.
Q03790	750	REPLACE BAD PRE-REGULATOR ON IRM 18 CHASSIS.	REPLACED PRE-REGULATOR AND ADJUSTED.
Q03934	1000	REPAIR 1A LPCI LOOP UPSTREAM STOP VALVE 1-1001-29A. THE STEM APPEARS DAMAGED.	FOUND STEM THREADS COMPACTED, BRASS SHAVINGS INSIDE STEM PROTECTOR, AND STEM BENT. REBUILT VALVE AND VERIFIED VALVE WAS OPERATIONAL.
Q04106	300	REPAIR SCRAM INLET VALVE 1-0305-126. ROD DRIFTS PAST WITH NO MANUAL SIGNAL.	ADJUSTED SEATING PRESSURE AND CHECKED SPRING TENSION.
Q04113	901	REPLACE FUSE AND COIL ON U1 PCI RELAY 595-137.	INSTALLED NEW RELAY AND HAD CONTROL ROOM VERIFY OPERATION.
Q04124	901	DURING RHR LOGIC TESTING, THE DELAY UNIT TIMED OUT TOO SLOWLY. INVESTIGATE PROBLEM WITH HFA RELAYS.	FOUND DEFECTIVE TIME DELAY UNIT. REMOVED RELAY FROM PANEL AND REPLACED WITH A NEW RELAY.

Q04225	300	REPAIR CRD CHARGING WATER VALVE 1-305-113-5015. UNABLE TO MANUALLY OPERATE HCE 50-15 113 VALVE.	FOUND EAR ON WEDGE WAS BROKEN OFF. REPLACED STEM AND DISC WAS REPACKED. INSTALLED VALVE AND RETORQUED BONNET.
Q79441	1000	REPAIR MISSING KEY WAY ON DECLUTCH HANDLE ON 1B RHR LOOP MIN FLOW VALVE 1-1001-18B.	MISSING KEY WAS REPLACED WITH A NEW KEY.
Q89607	2300	REPAIR HPCI TORUS SUCTION VALVE 1-2301-56. APPEARS TO BE BINDING.	INSTALLED NEW OPERATOR.
Q90017	300	REPAIR OR REPLACE 1A SCRAM DISCHARGE VOLUME LT 1-0302-109K HIGH SIDE SHUTOFF VALVE 1-0301-103A.	CHANGED OUT VALVE FOR NEW.
Q91119	5700	REPAIR SWITCH FOR THE HPCI ROOM COOLER.	FOUND THE SWITCH ACTUATOR HANGING UP. APPEARED SOMETHING WAS WARPED OR BENT IN THE COVER CAUSING THE SWITCH TO BIND. REPLACED SWITCH AND BOX WITH NEW.
Q96484	220	REPAIR LIMIT SWITCH ON 1B RECIRC LOOP UPSTREAM SAMPLE VALVE AO 1-0220-44.	INSTALLED NEW LIMIT SWITCHES.
Q96599	5400	REPAIR SJAE SUCTION VALVE 1-5401B (BLOWING AIR).	INSTALLED NEW SOLENOID VALVE. STROKED VALVE AND VERIFIED PROPER OPERATION WITH NO LEAKS.
Q97682	201	REPAIR U1 SHROUD ACCESS HOLE COVER PLATES.	VENDORS INSTALLED A NEW SHROUD ACCESS HOLE COVER.
Q98164	1300	REPLACE RCIC STEAM LINE HIGH FLOW SWITCH 1-1360-1A.	REPLACED SWITCH LIKE FOR LIKE.

Q02611	750	REPAIR IRM 16 CHASSIS, SPIKING HIGH.	RAN I/V CURVE, TDR TRACES, RESISTANCE TEST AND BOCK TEST. CALIBRATED IRM 11, 12, 13, 14, 16, AND 18 CHASSIS. ADJUSTED GAINS.
Q02814	7800	REPAIR MCC 18/19-5 CUBICLE B1 REACTOR WATER RECIRC PUMP SUCTION VALVE 1A.	FOUND WIRE FROM RED LIGHT TO TERMINAL 4 OFF. REPLACED WIRE AND VERIFIED PROPER LIGHT INDICATION.
Q03162	2300	REPLACE MISSING U-BOLTS ON HPCI LINE 1-2304-14" HANGER.	FOUND U-BOLT ON LEFT SIDE MISSING ALL NUTS THAT SECURE IT TO THE HANGER AND PIPE. INSTALLED 4 NUTS ON LEFT U-BOLT.
Q03265	7800	REPLACE BROKEN TERMINAL BLOCKS ON MCC 18/19-5 CUBICLE E1 RECIRC PUMP SUCTION VALVE 202-4B.	INSTALLED NEW TERMINAL BLOCK, LIFTED AND RELANDED WIRES, AND PUT CUBICLE BACK IN THE MCC.
Q03269	202	REPAIR HANDWHEEL ON U1 B RECIRC PUMP SUCTION VALVE 1- 0202-4B.	FOUND SET SCREW LOOSE ON THE HANDWHEEL. THE SET SCREW WAS TIGHTENED.
Q03287	1000	REPAIR 1A RHR TORUS SPRAY STOP VALVE 1-1001-37A. FAILED LLRT.	FOUND SCORE ACROSS THE SEAT OF DISC AND BODY SEAT. MACHINED AND POLISHED THE DISC AND BODY SEATS AND REASSEMBLED VALVE.
Q03288	220	REPAIR FEEDWATER HEADER "A" INBOARD CHECK VALVE 1-0220- 58A. FAILED LLRT.	REMOVED AND INSPECTED TRIM. FW1 AND FW2 WERE GROUND AND MACHINED. VALVE WAS REASSEMBLED.
Q03324	220	REPAIR FEEDWATER HEADER "A" 1ST OUTBOARD CHECK VALVE 1- 0220-62A.	FOUND THE O-RING LAND INADEQUATE FOR O-RING SEATING. WELD REPAIRED AND MACHINED THE VALVE BODY AND REASSEMBLED VALVE. PASSED LLRT.

Q03397	7800	REPLACE MAIN CONTACTOR COIL IN U1 A RPS MG SET MCC 18-2 CUBICLE E3.	FOUND CUBICLE E4 TO BE THE CORRECT CUBICLE. FOUND INTERNAL COIL TO HAVE SEVERAL CRACKS IN ITS CASING. VERIFIED OPERABILITY OF CONTACTOR. NO PROBLEMS FOUND. REPLACED COIL WITH NEW.
Q03427	302	REPLACE SCRAM DISCHARGE VOLUME LEVEL TRANSMITTER 1-0302-109D. OUT OF TOLERANCE.	REPLACED TRANSMITTER AND SENSOR BELLOWS ON LT 1-302-109D. ALSO REPLACED O-RING AND BACKUP O-RING ON BOTH SENSOR BELLOWS. REPLACED BUTT SPLICES IN LB BOX WITH QUALIFIED SPLICES.
Q03565	2300	RESET COLD LOAD SETTING OF HANGER 2305-W-102.	LOOSENED LOCK NUT AND SET COLD LOAD TO APPROXIMATELY 2900 LBS.
Q03566	8300	REPAIR MCC 1A CUBICLE I1 BREAKER.	FOUND BREAKER WITH BROKEN HANDLE. INSTALLED NEW BREAKER WITH NEW HARDWARE.
Q03611	202	REPACK U1 A RECIRC PUMP DISCHARGE VALVE 1-0202-5A.	FOUND PACKING LEAKING AND STEM HAD GROOVES THE FULL LENGTH OF THE STEM. RETORQUED PACKING GLANDS. AFTER TORQUING, SOME LEAKAGE WAS STILL PRESENT.
Q03910	1400	REPAIR LEAKING FLANGE ON 1A CORE SPRAY DISCHARGE HEADER SAFETY VALVE 1-1402-28A.	FOUND NUTS AT 60 FT-LBS. TIGHTENED NUTS SLIGHTLY.
Q03930	8300	REPLACE 250VDC BREAKER FOR MO-1-1301-25.	MODIFIED EXISTING BREAKER MOUNTING BRACKET AND MOUNTED NEW BREAKER. MOUNTED OLD BREAKER HANDLE. REPLACED BAD JUMPERS ON TERMINALS 1U, 1, 2, AND 6 WITH NEW JUMPERS. ADJUSTED MOUNTING HARDWARE ON CUBICLE DOOR TO MATCH UP WITH BREAKER HANDLE.

Q03980	300	REPAIR SCRAM PILOT AIR SOLENOID 1-0305-117. BLOWING AIR.	INSTALLED REBUILT VALVES. NO LEAKS FOUND.
Q03997	1000	REPAIR 1A CONTAINMENT SPRAY LOOP DOWNSTREAM STOP VALVE 1-1001-26A. FAILED LLRT.	REPLACED WORN BONNET BOLTING. REPACKED VALVE.
Q04034	8300	REPAIR CRACKED BREAKER HANDLE ON MCC 1A CUBICLE K2 FOR MO-1-2301-6.	REMOVED OLD BREAKER AND INSTALLED A NEW ONE.
Q04037	7800	REPAIR COIL IN MCC 18-1A CUBICLE D2 MAIN STEAM LINE OUTSIDE DRAIN VALVE.	REPLACED MOTOR STARTER. REPLACED LEFT CONTACTOR COIL WITH NEW.
Q04041	3000	REPAIR BENT HANGER BAR, MAIN STEAM 3001D-W-103.	CUT, THREADED, AND INSTALLED NEW HANGER ROD. INSTALLED NEW BOLTS.
Q04056	30	REPAIR MK483 PENETRATION, RHR SERVICE WATER VAULT.	FOUND LEAK ON TOP SIDE. TIGHTENED LINK SEAL BOLTS AT LOCATION OF LEAK UNTIL LEAK WAS STOPPED.
Q04057	30	REPAIR MK699 PENETRATION, RHR SERVICE WATER VAULT.	FOUND LEAKS ON INSIDE AND OUTSIDE SEALS. TIGHTENED LINK SEAL BOLTS AT LOCATION OF LEAKS UNTIL LEAKS WERE STOPPED.
Q04058	30	REPAIR MK470 PENETRATION, RHR SERVICE WATER VAULT.	FOUND LEAKS ON A FEW BOLTS. TIGHTENED LINK SEAL BOLTS AT LOCATION OF LEAKS UNTIL LEAKS WERE STOPPED.
Q04059	30	REPAIR MK507 PENETRATION, RHR SERVICE WATER VAULT.	FOUND LEAKS ON INSIDE AND OUTSIDE SEALS. TIGHTENED LINK SEAL BOLTS AT LOCATION OF LEAKS. OUTSIDE SEAL STILL LEAKS BUT IS ACCEPTABLE.

Q04069	1000	RESET CURRENT COLD LOAD SETTING ON U1 ISI HANGER 1012B-W-103, 1ST FLOOR DRYWELL.	UNLOADED SPRING TO 5978 LBS. LOADED SPRING TO APPROXIMATELY 6883 LBS.
Q04077	2300	REPAIR HPCI PUMP DOWNSTREAM TORUS SUCTION VALVE 1-2301-35.	POLISHED STEM. LAPPED WEDG' AND BODY SEATS. INSTALLED PACKING AND CARBON SPACER. TORQUED PACKING TO 90 LBS. INSTALLED NEW STEM NUT.
Q04139	2500	ADJUST FLOW CONTROLLER FI 1-2540-10A, ACAD DILUTION.	CALIBRATED SQUARE ROOTER AND FLOW INDICATOR. FOUND C1 AND C2 ON MAIN BOARD OF FC 1-2540-15A BAD. REPLACED WITH LIKE BOARD. REPLACED CAPACITORS C1 AND C2 WITH SAFETY RELATED COMPONENTS. REINSTALLED CONTROLLER.
Q04214	901	REPAIR "A" RPS MANUAL SCRAM PUSHBUTTON. STICKS.	INSTALLED NEW CONTACT BLOCK AND WIRED NEW BLOCK.
Q04321	2300	REPAIR U1 HPCI TESTABLE CHECK VALVE 1-2301-7. BLOWS BY.	INSTALLED NEW VALVE, RECONNECTED AIR LINES. CHECKED FOR PROPER OPERATION. OK.
Q04339	1000	REPAIR OUTBOARD BOOSTER PUMP SEAL ON 1D RHRSW PUMP.	FOUND WATER IN THE BEARING OIL AND PARTS OF BEARING IN THE OIL. DRAINED OIL. REMOVED BEARING AND OIL SEAL. REBUILT PUMP AND REINSTALLED.
Q04371	7800	REPAIR TERMINAL #1 ON MCC 18-1A.	CUT OLD LUG OFF TERMINAL #1 AND RELOGGED EACH LEAD WITH ITS OWN LUG AND RELOADED ON TERMINAL #1.
Q04372	220	REPAIR 1B DISCHARGE VACUUM BREAKER ELECTROMATIC SAFETY VALVE 1-0220-81B.	ALL PARTS WERE INSPECTED AND CLEANED.

Q04397	7800	REPAIR MCC 18-1A COMPARTMENT E4.	INSTALLED NEW MALE TERMINAL BLOCKS. DOCUMENTED ALL LEAD LIFTS AND LANDS. INSTALLED NEW FEMALE TERMINAL BLOCKS. DOCUMENTED ALL LEAD LIFTS AND LANDS.
Q04431	203	REPAIR OUTBOARD MSIV DC SOLENOID SEALTITE.	FOUND SEALTITE SMASHED. REPLACED SEALTITES AND TIGHTENED ALL CONNECTIONS.
Q04432	7200	REPLACE DEFECTIVE RMS-9 PROGRAMMER ON BREAKER 5018 BUS 19 CUBICLE 6D.	REPLACED RMS-9 WITH NEW.
Q04440	2300	REBRUSH HPCI MOTOR SPEED CHANGER MOTOR.	INSTALLED NEW BRUSHES.
Q04441	2300	REBRUSH HPCI TURBINE MOTOR GEAR UNIT.	INSTALLED NEW BRUSHES.
Q04463	300	REPAIR CONTROL ROD DRIVE E-7 FOUND LEAKING DURING HYDRO.	TORQUED ALL ACCESSIBLE BOLTS TO 375 FT-LBS. DRIPPING STILL PRESENT.
Q04464	203	REPAIR LEAK IN 1A MAIN STEAM LINE TARGET ROCK VALVE.	REPLACED GASKETS ON PILOT FLANGE AND RETORQUED TO 710 FT-LBS.
Q04469	263	REPAIR REACTOR LT 1-0263-58 HI SIDE EF CHECK VALVE 1-0263-2-13B.	DISASSEMBLED VALVE, CLEANED, AND INSPECTED INTERNALS. REASSEMBLED VALVE AND INSTALLED BACK IN SYSTEM.
Q04470	263	REPAIR LT 1-0646-A AND PT 1-0647-A HI SIDE EF CHECK VALVE 1-0263-2-19A.	DISASSEMBLED VALVE, CLEANED, AND INSPECTED INTERNALS. REASSEMBLED VALVE AND INSTALLED BACK IN SYSTEM.

Q04478	7800	REPAIR MCC 19-1 CUBICLE H2 REACTOR WATER CLEANUP PUMP 1B BREAKER. BREAKER WILL NOT GO TO OFF.	REMOVED CUBICLE, REMOVED OLD BREAKER. REPLACED WITH NEW BREAKER. PUT CUBICLE BACK IN MCC 19-1.
Q04479	1000	REPAIR RHRSW FROM 1 RHR HEAT EXCHANGER FLOW CONTROL VALVE 1-1001-5A. VALVE WILL NOT STROKE.	FOUND PIECE OF METAL IN VALVE BODY BELOW DISC GUIDE. SEAT AND DISC WERE SCRATCHED. REPLACED LIQUIDTITE CONNECTOR ON SEALTITE AND RETURNED SEALTITE BACK TO VALVE. REPACKED AND REASSEMBLED VALVE.
Q04494	207	REPLACE MISSING BOLT FOR REACTOR VESSEL DRAIN TO REACTOR WATER CLEANUP PIPING SUPPORT 1-207-2.	INSTALLED NEW BOLT AND NUTS ON THE SPRING CAN PIPE SUPPORT.
Q04496	4700	REPAIR MISSING THREADED RODS FORM MSIV ACCUMULATOR PIPE HANGERS.	MANUFACTURED AND INSTALLED NEW HANGER RODS.
Q04499	220	REPAIR/REPLACE CHECK VALVE V-1, 2A MSIV AIR SUPPLY.	FOUND VALVE EXTERIOR CORRODED AND DISC RUBBER VERY HARD. BRAZED IN NEW CHECK VALVE.
Q04500	261	REPAIR D/P INDICATOR SWITCH FOR MAIN STEAM LINE FLOW, 1-0261-2H.	REPLACED THE MANIFOLD AND CALIBRATED THE INDICATOR.
Q04513	7800	ELIMINATE WIRES AND LUG ON MCC 18-1A COMPARTMENT F4 INTERNAL WIRING.	CORRECTED WIRING.
Q04518	7800	REWIRE MCC 18-1A CUBICLE A3 MAIN STEAM LINE "A" DRAIN VALVE.	RELUGGED TWO WIRES AND RELANDED ON TERMINAL ONE.

Q04542	2300	ADJUST U1 HPCI MGU HIGH SPEED STOP LIMIT SWITCHES 18A AND 18C.	ADJUSTED LIMIT SWITCH AND TIGHTENED ALL HARDWARE.
Q04543	7800	REPAIR BROKEN CONTROL SWITCH ON MCC 19-1 CUBICLE D5 BREAKER.	INSTALLED NEW CIRCUIT BREAKER.
Q04545	1600	REPAIR DRYWELL TO TORUS RING HEADER VACUUM BREAKER SOLENOID AO 1-1601-32A.	FOUND TWO BOLTS MISSING FROM HOFFMAN BOX. REPLACED TWO BOLTS AND TORQUED TO SPECS.
Q04604	901	REPAIR BROKEN INSULATING DIVIDERS ON TERMINAL BOARD IN CONTROL ROOM PANEL 901-4.	FOUND BROKEN TERMINAL BLOCK AA AT POINTS 30, 31, 32, 86, 9, 14, 63, AND 35. REPLACED ALL BROKEN TERMINAL BLOCKS. VERIFIED CONTINUITY.
Q04620	7800	REPAIR MCC 19-1 CUBICLE H1 ROOM COOLER BREAKER CONTACTS.	FOUND AUXILIARY CONTACTS HANGING UP. CLEANED AND LUBED AUX CONTACT SIDING ARMS.
Q04642	300	REPAIR NITROGEN CYLINDER ACCUMULATOR 1-0305-128.	REMOVED ACCUMULATOR. REMOVED OLD O-RINGS AND CLEANED ALL SURFACES. INSTALLED NEW LUBRICATED O-RINGS ON NEW ACCUMULATOR. INSTALLED NEW ACCUMULATOR.
Q04661	2300	REPAIR/REPLACE HPCI GLAND SEAL CONDENSER PUMP CHECK VALVE 1-2301-76.	CLEANED DISC, STEM, VALVE BODY, BONNET, AND GASKET MATERIAL. VERIFIED CHECK VALVE OPERATIONAL.
Q04669	2300	REPAIR OIL LEAK ON FLEX LINE ON HPCI PUMP 1-2302.	FOUND COUPLING SIDE OF HOSE CRACKED. REMOVED, CLEANED, AND REPLACED WITH NEW HOSE.

Q04670	2300	REPAIR HPCI PROCESS FLOW INDICATING CONTROLLER 1-2340-1.	NO PROBLEM FOUND.
Q04687	8300	ADJUST RELAY IN CUBICLE 250VDC BUS 1A HPCI TURBINE GLAND SEAL CONDENSER. WOULD NOT PICKUP.	CHECKED PICKUP VOLTAGES. NO ADJUSTMENT MADE.
Q04741	2400	INVESTIGATE/REPAIR 1A CAM TORUS OXYGEN SAMPLE RECOVERER.	NO PROBLEM FOUND. SUSPECT THE LOCAL PANEL WAS LEFT IN SPAN MODE AFTER COMPLETION OF QCCP 1200-4 ON 12/11/92 BY CHEMISTRY.
Q04746	300	TROUBLESHOOT AND REPAIR SCRAM SOLENOID 118 VALVE.	NO PROBLEM FOUND. REMOVED, REBUILT, AND REINSTALLED VALVES ANYWAY.
Q04978	1000	REPAIR INDICATION ON 1B RHR PUMP TORUS SUCTION VALVE.	REMOVED CUBICLE A3 AND CLEANED STAB AND STAB ASSEMBLY.
Q78596	2300	FIX BROKEN PLUG CONNECTION AT LOCAL JUNCTION BOX ON WEST END OF HPCI TURBINE.	FOUND BROKEN PLUG ON SEALTITE END. REPLACED WITH NEW STYLE PLUG.
Q78817	7800	REPAIR MCC 19-1 CUBICLE C2 U1 RBCCW HEADER ISOLATION VALVE MOTOR STARTER.	REPLACED BOTH STARTER COILS WITH E.Q. STARTER COILS.
Q79531	263	INVESTIGATE/REPAIR GROSS FAILURE LIGHT AND ANALOG ALARM ON ATWS MASTER TRIP UNIT.	CALIBRATED MASTER TRIP UNIT AND REINSTALLED IN PANEL.
Q82846	2300	DISASSEMBLE AND INSPECT U1 HPCI TURBINE INLET VALVE 1-2301-3.	DISASSEMBLED, REMACHINED VARIOUS PARTS OF THE VALVE, REBUILT VALVE, AND REINSTALLED VALVE.

Q93436	8300	CLEAN CORROSION FORM BUS BAR CONNECTION INSIDE 250VDC BREAKER FOR HPCI TURBINE AUXILIARY OIL PUMP.	CLEANED AND TAPED BUS BARS.
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UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	WORK REQUESTED	WORK PERFORMED
Q00662	5400	REPAIR IN LINE OILER LEAK ON SJAE SUCTION VALVE 2-5401B.	CHANGED OUT OLD OILER FOR NEW AND REFILLED WITH OIL.
Q03513	300	INVESTIGATE DRIFTING SCRAM DISCHARGE VOLUME FLOW CONTROL INDICATING LEVEL SWITCHES 2-302-82L.	REPLACED AND CALIBRATED THE CIRCUIT BOARD. VERIFIED OPERATION. SUSPECT BAD RELAY.
Q05110	1600	REPAIR SOLENOID ON 2-1601-56 VALVE.	REPLACED THE OLD SOLENOID WITH A NEW ONE.
Q05112	203	DURING RPS BUS TRANSFER, THE 2A MSIV WENT CLOSED. INVESTIGATE SUSPECTED FAULTY DC SOLENOID.	REPLACED MANIFOLD ON VALVE JUNCTION BOX. CHECKED VOLTAGE OF ALL THREE COILS. ALL READINGS WERE IN RANGE.
Q05181	760	REPAIR IRM 11.	INSTALLED NEW PREAMPLIFIER.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
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There were no Licensee Event Reports for Unit 1 for this reporting period.

UNIT 2

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
93-001	01/07/93	Rx SCRAM when MSIV went close during power transfer on RP?
93-002	01/13/93	HPCI MSL failure to run back to LSS
93-003	01/14/93	Loss of light indication for HPCI controls
93-004	01/15/93	Missed Tech Spec Surveillance for recombiner temperature
93-005	01/29/93	Rx SCRAM from high Rx

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE February 8, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 010193
1. REPORTING PERIOD: 2400 013193 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):
5. NUMBER OF HOURS REACTOR WAS CRITICAL THIS MONTH' YR TO DATE CUMULATIVE
6. REACTOR RESERVE SHUTDOWN HOURS 0.0 0.0 3421.9
7. HOURS GENERATOR ON LINE 744.0 744.0 139135.6
8. UNIT RESERVE SHUTDOWN HOURS 0.0 0.0 909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)..... 1816022.4 1816022.4 299397846.2
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)..... 592684.0 592684.0 97011774.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)..... 567894.0 567894.0 91443406.0
12. REACTOR SERVICE FACTOR..... 100.00 100.00 78.68
13. REACTOR AVAILABILITY FACTOR..... 100.00 100.00 80.55
14. UNIT SERVICE FACTOR 100.00 100.00 76.28
15. UNIT AVAILABILITY FACTOR 100.00 100.00 76.78
16. UNIT CAPACITY FACTOR (Using MDC) 99.26 99.26 65.19
17. UNIT CAPACITY FACTOR (Using Design MWe) 96.74 96.74 63.54
18. UNIT FORCED OUTAGE RATE 0.00 0.00 5.79
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE February 8, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 010193
1. REPORTING PERIOD: 2400 013193 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY: 769
DESIG ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):
5. NUMBER OF HOURS REACTOR WAS CRITICAL THIS MONTH YR TO DATE CUMULATIVE
579.30 579.30 139754.55
6. REACTOR RESERVE SHUTDOWN HOURS 0.0 0.0 2985.80
7. HOURS GENERATOR ON LINE 544.50 544.50 136185.95
8. UNIT RESERVE SHUTDOWN HOURS 0.0 0.0 702.90
9. GROSS THERMAL ENERGY GENERATED (MWH)..... 1306188.00 1306188.00 293963597.20
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)..... 429389.00 429389.00 94454650.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)..... 410168.00 410168.00 89440911.00
12. REACTOR SERVICE FACTOR..... 77.86 77.86 77.28
13. REACTOR AVAILABILITY FACTOR..... 77.86 77.86 78.93
14. UNIT SERVICE FACTOR 73.19 73.19 75.31
15. UNIT AVAILABILITY FACTOR 73.19 73.19 75.70
16. UNIT CAPACITY FACTOR (Using MDC) 71.69 71.69 64.32
17. UNIT CAPACITY FACTOR (Using Design MWe) 69.87 69.87 62.69
18. UNIT FORCED OUTAGE RATE 26.81 26.81 7.88
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE February 8, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH January 1993

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>784</u>
2.	<u>784</u>
3.	<u>760</u>
4.	<u>724</u>
5.	<u>746</u>
6.	<u>782</u>
7.	<u>756</u>
8.	<u>782</u>
9.	<u>782</u>
10.	<u>767</u>
11.	<u>779</u>
12.	<u>747</u>
13.	<u>778</u>
14.	<u>779</u>
15.	<u>782</u>
16.	<u>727</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>771</u>
18.	<u>784</u>
19.	<u>784</u>
20.	<u>757</u>
21.	<u>752</u>
22.	<u>742</u>
23.	<u>709</u>
24.	<u>707</u>
25.	<u>755</u>
26.	<u>761</u>
27.	<u>774</u>
28.	<u>727</u>
29.	<u>781</u>
30.	<u>783</u>
31.	<u>783</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE February 8, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH January 1993

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>788</u>
2.	<u>790</u>
3.	<u>786</u>
4.	<u>762</u>
5.	<u>787</u>
6.	<u>786</u>
7.	<u>328</u>
8.	<u>- 8</u>
9.	<u>- 8</u>
10.	<u>- 8</u>
11.	<u>- 8</u>
12.	<u>- 8</u>
13.	<u>201</u>
14.	<u>729</u>
15.	<u>794</u>
16.	<u>794</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>761</u>
18.	<u>783</u>
19.	<u>671</u>
20.	<u>669</u>
21.	<u>752</u>
22.	<u>795</u>
23.	<u>796</u>
24.	<u>793</u>
25.	<u>793</u>
26.	<u>764</u>
27.	<u>774</u>
28.	<u>788</u>
29.	<u>448</u>
30.	<u>- 8</u>
31.	<u>- 3</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One

DATE February 8, 1993

REPORT MONTH January 1993

COMPLETED BY M. Benson

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
93-00									There were no power reductions of greater than 20% over a 24 hour period for the month of January 1993.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two

DATE February 8, 1993

COMPLETED BY M. Benson

TELEPHONE 309-654-2241

REPORT MONTH January 1993

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
93-01	01/07/93	F	142.5	A	3	04-02-93-001			Auto Reactor SCRAM - RPS Power Transfer
93-02	01/29/93	F	57.0	H	3	04-02-93-005			Auto Reactor SCRAM - High Pressure

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
 PERFORMED ON UNIT 1 & 2 CONTROL
 ROD DRIVES, FROM 01/01/93 TO 01/31/93

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		
		0.375	0.900	2.00	3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
01/28/93	1	0.30	0.67	1.42	2.15	H-12 2.15	For Accumulator Replacement

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: 01 Reload: 12 Cycle: 13
2. Scheduled date for next refueling shutdown: 3-14-94
3. Scheduled date for restart following refueling: 6-13-94
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

NOT AS YET DETERMINED

5. Scheduled date(s) for submittin: proposed licensing action and supporting information:

NOT AS YET DETERMINED

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.

7. The number of fuel assemblies.

- a. Number of assemblies in core: 724
- b. Number of assemblies in spent fuel pool: 1557

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

- a. Licensed storage capacity for spent fuel: 3657
- b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q2 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 03/06/93
3. Scheduled date for restart following refueling: 06/05/93
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

NO
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

NO SUBMITTALS REQUIRED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 2439
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center