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Writer's Direct Dial Number:  
February 11, 1993  
C321-93-2056

U. S. Nuclear Regulatory Commission  
Att: Document Control Desk  
Washington, DC 20555

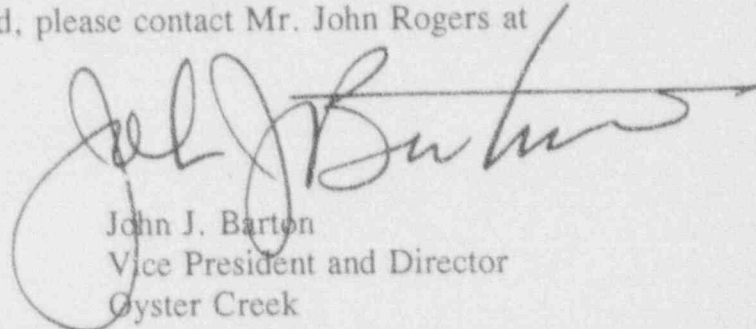
Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Request for Relief from ASME Section XI, IWA-5250(a)(2)

Dear Sir:

Reference: Letter No. C321-93-2041, Barton to USNRC dated February 4, 1993

On February 4, 1993, GPU Nuclear requested relief from the provisions of ASME Section XI, IWA-5250(a)(2). Subsequent to that letter, a teleconference was held between GPU Nuclear and several sections of the USNRC. This letter is being written to provide additional information to the USNRC as requested during that teleconference call.

The requested information is provided in Attachment I to this letter. If any additional information or assistance is required, please contact Mr. John Rogers at 609.971.4893.



John J. Barton  
Vice President and Director  
Oyster Creek

JJB/JJR

cc: Administrator, Region I  
Senior Resident Inspector  
Oyster Creek NRC Project Manager

9302170280 930211  
PDR ADOCK 05000219  
P PDR

## ATTACHMENT I

1. The relief requested in the referenced letter is specific to the leaking bolted connections which were discovered at the Oyster Creek Nuclear Generating Station during the NSSS Leak test which was conducted near the conclusion of the 14R refueling outage.
2. As described in request A in the referenced letter, the specific components for which relief is requested are:

Recirculation Pump A, Body to Casing Seal  
Recirculation Pump B, Body to Casing Seal  
Recirculation Pump C, Body to Casing Seal

Recirculation Valve V-37-11

Main Steam Isolation Valve V-1-7  
Main Steam Isolation Valve V-1-8

The specific components listed above were first inspected during the first days of the outage, while the plant was at normal temperature and pressure. No leaks were identified at that time. An additional inspection for leakage will be conducted during the 1000 psig inspection during the restart from the current refueling outage.

3. With respect to Request B in the referenced letter, the specific components for which relief is requested are:

Control Rod Drives:

14-19	22-51
14-47	46-39
22-35	42-43

The specific components listed above were first inspected during the first days of the outage, while the plant was at normal temperature and pressure. No leaks were identified at that time. Oyster Creek has not experienced any cracking or corrosion on any CRD vessel flange bolts. The bolts used by Oyster Creek are of a lower hardness than the bolts which have exhibited cracking at other plants.