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February 10, 1993

Document Control Desk
U.S. NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555

Gentlemen:

DOCKET 50-301
CYCLE 19 RELOAD
POINT BEACH NUCLEAR PLANT, UNIT 2

Point Beach Nuclear Plant, Unit 2, was shut down for refueling at the end of Cycle 18 operation on September 26, 1992. Cycle 18 burn-up was approximately 10,772 MWD/MTU. Unit 2 began operation for Cycle 19 on November 18, 1991, following a 7-week refueling and maintenance outage.

Reload Region 21 for Unit 2 Cycle 19 operation contains 28 Westinghouse 14 x 14 upgraded Optimized Fuel Assemblies (OFA). Upgraded OFAs include removable top nozzles, debris filter bottom nozzles, and extended burn-up geometry. This is the fourth reload region of upgraded OFA fuel inserted into the Unit 2 core. The use of upgraded OFA fuel in both Point Beach Nuclear Plant units was reviewed and approved, as reported in the NRC Nuclear Safety Evaluation Report issued on May 8, 1989, in support of License Amendment No. 123 (Technical Change Request 127) for Unit 2.

Additional changes approved under License Amendment No. 123 and implemented for Cycle 19 are increased allowable core power peaking factors, thimble plug removal, the use of hafnium peripheral power suppression assemblies, integral fuel burnable absorber assemblies, and axial blankets. License Amendment No. 126, which approved the removal of the $f(\Delta I)$ function from the overpower delta-T setpoint calculation, was also considered in the safety evaluations performed for Cycle 19.

The mechanical and thermal-hydraulic designs for the Unit 2 Cycle 19 reload core are similar to those of previously reviewed and accepted reload designs containing OFA fuel. Two minor mechanical changes to the new fuel are: (1) top and bottom anti-sag grids, and (2) a longer two-piece bottom grid insert. These

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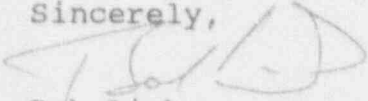
changes have been evaluated by Westinghouse, and it has been determined that these changes do not affect the previous mechanical and thermal-hydraulic designs.

This core is designed to operate under nominal design parameters and the approved Technical Specifications, including those provided with License Amendment Nos. 123 and 126 for Unit 2. For those postulated accidents presented in the FSAR which could be affected by the reload core, re-evaluation has demonstrated that the results of the postulated events are within allowable limits. The reload core meets the current $F_Q \times P$ limit of 2.5 and the current FAH limit of 1.70.

In accordance with past practice, the Westinghouse reload safety evaluation report relies on previously reviewed and accepted analyses as reported in the FSAR, in the upgraded OFA safety reports, and in earlier reload cycle safety evaluation reports. The reload safety report for Unit 2 Cycle 19 concludes that no unreviewed safety questions, as defined in 10 CFR 50.59, are involved in the operation of Unit 2 during Cycle 19. This 10 CFR 50.59 evaluation has been reviewed and approved by the Manager's Supervisory Staff. Verification of the core design has been performed by means of the standard start-up physics tests conducted at the beginning of this cycle.

Please contact us, if you have any questions regarding the Cycle 19 reload design or operation.

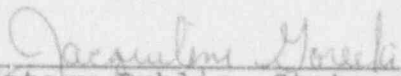
Sincerely,


Bob Link
Vice President
Nuclear Power

CAC/akf

cc: NRC Resident Inspector
NRC Regional Administrator, Region III

Subscribed and sworn to before me
this 10th day of February, 1993.


Notary Public, State of Wisconsin

My Commission expires 10-27-96.