

LICENSEE EVENT REPORT (LER)

| | | | | | | | | | | | | | | | | |
|--|--------|--|----------------|-------------------|-----------------|-------------------|-----------------|-----------|----------------------|---|--|-------------------------------|--|--|-----|------|
| FACILITY NAME (1) Quad-Cities Nuclear Power Station, Unit 2 | | | | | | | | | | DOCKET NUMBER (2) 0 5 0 0 0 2 6 5 | | | | PAGE (3) 1 OF 0 3 | | |
| TITLE (4) Unit 2 Main Steam Isolation Valves Failed Local Leak Rate Tests | | | | | | | | | | | | | | | | |
| EVENT DATE (5) | | | LER NUMBER (6) | | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | |
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES NA | | | DOCKET NUMBER(S) 0 5 0 0 0 | | | | |
| 0 3 | 1 8 | 8 5 | 8 5 | 0 0 6 | 0 1 0 | 6 2 | 4 8 | 5 | | | | 0 5 0 0 0 | | | | |
| OPERATING MODE (9) 2 | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11) | | | | | | | | | | | | | | |
| POWER LEVEL (10) 0 0 0 | | 20.402(b) | | | | 20.408(a) | | | | 50.73(a)(2)(iv) | | | | 73.71(b) | | |
| | | 20.408(a)(1)(i) | | | | 50.73(a)(1) | | | | 50.73(a)(2)(v) | | | | 73.71(c) | | |
| | | 20.408(a)(1)(ii) | | | | 50.73(a)(2) | | | | 50.73(a)(2)(vi) | | | | OTHER (Specify in Abstract below and in Text, NRC Form 365A) | | |
| | | 20.408(a)(1)(iii) | | | | 50.73(a)(2)(i) | | | | 50.73(a)(2)(vii)(A) | | | | | | |
| | | 20.408(a)(1)(iv) | | | | X 50.73(a)(2)(ii) | | | | 50.73(a)(2)(vii)(B) | | | | | | |
| | | 20.408(a)(1)(v) | | | | 50.73(a)(2)(iii) | | | | 50.73(a)(2)(viii) | | | | | | |
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | | | | | | | | |
| NAME Nicos P. Digrindakis, Technical Staff (extension 194) | | | | | | | | | | TELEPHONE NUMBER AREA CODE 3 0 9 6 5 4 1 - 2 2 4 1 | | | | | | |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | | | | | | | |
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | | | | | | |
| B | J M | I S V C | 6 8 4 | Y | | | | | | | | | | | | |
| | | | | | | | | | | | | | | | | |
| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | | | | | | | EXPECTED SUBMISSION DATE (15) | | MONTH | DAY | YEAR |
| YES (If yes, complete EXPECTED SUBMISSION DATE) | | | | | | | | | | | | X NO | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewriter lines) (16)

The following is a supplementary report submitted to document the cause and corrective action for the Local Leak Rate Test failures of Main Steam Isolation Valves during End of Cycle Seven Refueling at Quad-Cities Unit Two.

On March 18, 1985, two Main Steam Isolation Valves (SB) leaked in excess of the 11.5 SCFH limit given in Technical Specification 3.7.A.2.a.3. The excessive leakage for valves AO 2-203-2B and AO 2-203-2D were identified during Local Leak Rate Testing performed while Unit 2 was shutdown for the End of Cycle Seven Refueling and Maintenance Outage.

The cause of the excess leakage was valve seating surface wear. The valves were repaired by lapping the main seats, and by replacing the main disc, pilot seats, and the pilot discs. After the repairs, each valve was Local Leak Rate Tested and the results indicated a measured leak rate of 4.6 SCFH for AO 2-203-2B and 9.2 SCFH for AO 2-203-2D.

This report is submitted to you in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(ii) to document the cause and corrective actions associated with the Local Leak Rate Test failures experienced on the Unit 2 Main Steam Isolation Valves.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

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|---|--|----------------|----------------------|--------------------|----------|----|-----|
| FACILITY NAME (1) Qua. Cities Nuclear Power Station, Unit 2 | DOCKET NUMBER (2) 0 5 0 0 0 2 6 5 8 5 | LER NUMBER (8) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | — 0 | 0 6 | — 0 1 | 0 2 | OF | 0 3 |

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Event Description

On March 18, 1985, Unit 2 was shutdown for the End of Cycle Seven Refueling and Maintenance Outage. At 0825 hours, while performing the Main Steam Isolation Valves (SB) Local Leak Rate Test, QTS 100-3, valves AO 2-203-2B and AO 2-203-2D were found to leak in excess of the 11.5 standard cubic feet per hour (SCFH) limit specified in Technical Specification 3.7.A.2.a.3. The leakage for the individual valves are as follows:

| <u>Valve</u> | <u>Leakage</u> |
|--------------|----------------|
| AO 2-203-2B | 16.2 SCFH |
| AO 2-203-2D | 191.2 SCFH |

The through-line leakages of the 'B' and 'D' steam lines are 0.00 SCFH due to the fact that the inboard valves AO 2-203-1B and AO 2-203-1D are leak-tight as shown by leak tests. Therefore, the safety implications of this occurrence were minimal.

This report is submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(ii).

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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|---|---|----------------|----------------------|--------------------|----------|--|--|
| FACILITY NAME (1) Quad-Cities Nuclear Power Station, Unit 2 | DOCKET NUMBER (2) 0 5 0 0 0 2 6 5 8 5 - 0 0 6 - 0 1 0 2 OF 0 3 | LER NUMBER (8) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | | | | | | |

TEXT (If more space is required, use additional NRC Form 385A's) (17)

Cause

The cause of this occurrence is valve seating surface wear. Both the main seat and disc and the pilot seat and disc showed signs of wear. The major factor contributing to this wear is the presence of high pressure steam flowing through the system during operation.

Corrective Action

The following list summarizes the repairs required for each valve:

| <u>Valve</u> | <u>Description of Repairs</u> |
|--------------|---|
| AO 2-203-2B | Lapped main seat, replaced main disc, pilot seat, and pilot disc. |
| AO 2-203-2D | Lapped main seat, replaced main disc, pilot seat, and pilot disc. |

After the above repairs were completed, each valve was Local Leak Rate tested. The results of these tests are listed below:

| <u>Valve</u> | <u>Leak Rate</u> |
|--------------|------------------|
| AO 2-203-2B | 4.6 SCFH |
| AO 2-203-2D | 9.2 SCFH |



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NJK-85-176

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

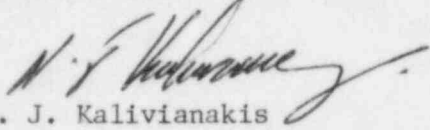
Reference: Quad-Cities Nuclear Power Station
Docket Number 50-265, DPR-30, Unit Two

Enclosed please find Licensee Event Report (LER) 85-006, Revision 1, for Quad-Cities Nuclear Power Station.

This supplemental report is submitted to you in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(ii) to document the cause and corrective actions associated with the Local Leak Rate Test failures experienced on the Unit Two Main Steam Isolation Valves.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION


N. J. Kalivianakis
Station Manager

NJK:BRS/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center
NRC Region III

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