



CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

362 INDIAN HOLLOW ROAD • EAST HAMPTON, CT 06424-3099

February 15, 1993  
Re: Technical Specification 6.9.1.8  
Docket No. 50-213

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements of Technical Specification 6.9.1.8, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 93-02 covering operations for the period January 1, 1993 to January 31, 1993 is hereby forwarded.

Very truly yours,

John P. Stetz  
Vice President  
Haddam Neck Station

JPS/va

- cc: (1) Regional Administrator, Region 1  
U. S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406
- (2) William Raymond  
Sr. Resident Inspector  
Connecticut Yankee

170029

Connecticut Yankee Atomic Power Company

Haddam Neck Plant

Haddam, Connecticut

Monthly Operating Report No. 93-02

For The Month of

January 1993



CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR#1 • BOX 127E • EAST HAMPTON, CT 06424-9341

February 1, 1993

The following is the Summary of Plant Operations for January 1992:

The plant was operated at 100% power until January 17th at 0025 hours. A power reduction to 65% was performed to conduct a routine turbine stop valve stroke test. The test was successfully completed at 0300 hours. A power increase was initiated and the plant was returned to 100% power at 0650 hours.

The plant was operated at 100% power for the remainder of the month of January.

CONNECTICUT YANKEE  
REACTOR COOLANT DATA  
MONTH: January 1993

REACTOR COOLANT ANALYSIS

	MINIMUM	AVERAGE	MAXIMUM
pH @ 25° C	6.59	6.77	6.91
CONDUCTIVITY ( $\mu$ mhos/cm)	10.67	12.95	15.97
Chlorides (ppm)	<0.050	<0.050	<0.050
Dissolved Oxygen (ppb)	<5	<5	<5
Boron (ppm)	298	338	384
Lithium (ppm)	0.719	0.908	1.125
Total Act. ( $\mu$ Ci/ml)	0.260	0.374	0.419
I-Ratio	0.67	0.72	0.97
I 131 ( $\mu$ Ci/ml)	1.74E-03	1.89E-03	2.34E-03
Crud (mg/l)	<1	<1	<1
TRITIUM ( $\mu$ Ci/mL)	2.66	3.50	4.23
Hydrogen (cc/Kg)	24.40	26.18	27.79

Aerated Liquid Waste Processed (Gallons): 1.48E+005  
Waste Liquid Processed through Boron Recovery (Gallons): 5.20E+004  
Average Primary Leak Rate (Gallons per Minute): .368  
Primary to Secondary Leak Rate (Gallons per Minute): 5.92E-03

**MAINTENANCE DEPARTMENT**  
Report Month January 1993

System or Component	MALFUNCTION		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide For Reactor Safety During Repair
	Cause	Result			
	There were no reportable items for the Maintenance Department for January 1993				

## I&C DEPARTMENT

Report Month: January 1993

System or Component	MALFUNCTION		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide For Reactor Safety During Repair
	Cause	Result			
There were no reportable items for the I&C Department for the month of January 1993					

## UNIT SHUTDOWNS AND POWER REDUCTION

Docket No: 50-213

Unit Name: Connecticut Yankee

Date: February 15, 1993

Completed By: K. W. Emmons

Telephone: (203) 267-3654

Report Month: January 1993

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting down Reactor	LER Report #	System Code	Component Code	Cause and Corrective Action to Prevent Recurrence
There were no reportable shutdowns or power reduction in January 1993									

### TYPE

F Forced  
S Scheduled

### REASON

A Equipment Failure  
B Maintenance or Test  
C Refueling  
D Regulatory Restriction  
E Operator Training  
F Administrative  
G Operator Error  
H Other (Explain)

### METHOD

1 Manual  
2 Manual Scram  
3 Automatic Scram  
4 Continued  
5 Reduced Load  
9 Other

### SYSTEM & COMPONENT

Exhibit F & H - Instructions for Preparation of  
Data Entry Sheets  
Licensee Event Report (LER)  
File (NUREG-0161)



# NRC OPERATING STATUS REPORT

Haddam Neck

1. Docket: 50-213
2. Reporting Period: 01/93      Outage + On-line Hours: 0.0 + 744.0 = 744.0
3. Utility Contact: M. P. Bain (203) 267-3635
4. Licensed Thermal Power (MWt): 1825
5. Nameplate Rating (Gross MWe):  $667 \times 0.9 = 600.3$
6. Design Electrical Rating (Net MWe): 582
7. Maximum Dependable Capacity (Gross MWe): 586.9
8. Maximum Dependable Capacity (Net MWe): 560.1
9. If changes occur above since last report, reasons are: NONE
10. Power level to which restricted, if any (Net MWe): N/A
11. Reasons for restriction, if any: N/A

	MONTH	YEAR-TO-DATE	CUMULATIVE
12. Report period hours:	744.0	744.0	219,912.0
13. Hours reactor critical:	744.0	744.0	175,550.4
14. Reactor reserve shutdown hours:	0.0	0.0	1,285.0
15. Hours generator on-line:	744.0	744.0	169,031.2
16. Unit reserve shutdown hours:	0.0	0.0	398.0
17. Gross thermal energy generated (Mwth):	1,347,687.0	1,347,687.0	292,260,459.0 *
18. Gross electrical energy generated (MWeH):	451,327.0	451,327.0	95,1909.0 *
19. Net electrical energy generated (MWeH):	431,095.0	431,095.0	90,992,296.6 *
20. Unit service factor:	100.0	100.0	76.9
21. Unit availability factor:	100.0	100.0	77.0
22. Unit capacity factor using MDC net:	103.5	103.5	75.1
23. Unit capacity factor using DER net:	99.6	99.6	71.1
24. Unit forced outage rate:	0.0	0.0	5.6
25. Forced outage hours:	0.0	0.0	10,078.5

26. Shutdowns scheduled over next 6 months (type, date, duration): 5/15/93-7/27/93 Refueling

27. If currently shutdown, estimated startup date: N/A

\* Cumulative values from the first criticality (07/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).



### Refueling Information Request

1. Name of facility  
Haddam Neck
2. Scheduled date for next refueling shutdown.  
May 15, 1993
3. Scheduled date for restart following refueling.  
July 27, 1993
4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?  
Yes  
(b) If answer is yes, what, in general, will these be?  
Technical Specification change to remove three loop specifications and failed fuel rod limit specification.  
(c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?  
n/a  
(d) If no such review has taken place, when is it scheduled?  
n/a
5. Scheduled date(s) for submitting proposed licensing action and supporting information.  
March 1993
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.  
n/a
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.  
(a) 157 (b) 757
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.  
1168
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.  
1998