

May 21, 1985

MEMORANDUM FOR: Charles E. Norelius, Director
Division of Reactor Projects
Region III

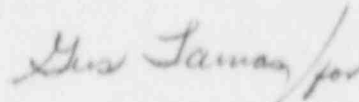
FROM: Hugh L. Thompson, Jr., Director
Division of Licensing
Office of Nuclear Reactor Regulation

SUBJECT: ISI AND IST PROGRAMS AT QUAD CITIES STATION,
UNITS 1 AND 2 (TIA 83-90)

In your memorandum of April 22, 1983 you requested NRR to clarify what ISI/IST program the license should follow, in that the first 10-year program period had expired and the second 10-year program had been submitted by the licensee but had not yet been approved by NRR. Task Interface Agreement (TIA) 83-90 was issued July 9, 1983 to task NRR with this undertaking.

By letter of March 20, 1985 (enclosed) following our preliminary review of the licensee's program as proposed in letters dated February 17, 1983, September 7, 1984 and January 1, 1985, we instructed the licensee to follow the licensee-proposed program, subject to conditions described in our letter on the subject, until our review is complete and final approval is given.

This complete NRR action on TIA 83-90.



Hugh L. Thompson, Jr., Director
Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As stated

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

R. BLVAN

March 20, 1985

Docket Nos. 50-254/265

Mr. Dennis L. Farrar
Director of Nuclear Licensing
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: INSERVICE INSPECTION AND TESTING PROGRAM -
SECOND 10-YEAR INTERVAL

Re: Quad Cities Nuclear Power Station, Units 1 and 2

By letters dated February 17, 1983, September 7, 1984 and January 9, 1985, you submitted a proposed inservice inspection and testing program description and requests for relief from selected ASME Code requirements pursuant to 10 CFR 50.55a(g). Although we have not completed our detailed review of your submittal, our preliminary review makes clear to us that your proposed program to implement those ASME Code requirements that you have found to be practical would increase the scope of inservice inspection and testing for your facility beyond that currently required by your Technical Specifications. We have concluded that this upgrading of your inservice inspection and testing program will further enhance safety.

Based on our preliminary review, we agree with the determination that it is impractical within the limitations of design, geometry and materials of construction of component, for you to meet certain of the specified ASME Code requirements and that imposition of those requirements would result in hardships or unusual difficulties without a compensating increase in the level of quality of safety. Therefore, pursuant to 10 CFR 50.55a(g)(6)(i), we hereby grant relief until the staff has completed its detailed review or until December 31, 1985, whichever comes first, from those inservice inspection and testing requirements of the ASME Code that you have requested. Moreover, since the scope of the inservice inspection and testing will be increased by your proposed program, and the granting of this relief is based only on the impracticality of selected ASME Code requirements, we have determined that the relief granted neither increases the probability or consequences of accidents previously considered nor decreases safety margins and that, therefore it does not involve a significant hazards consideration. Therefore, you are authorized to, and should proceed to, implement your proposed program (except where your current Technical Specifications are more restrictive).

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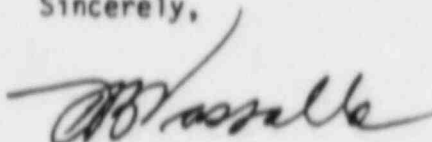
Mr. Dennis L. Farrar

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During the period between now and the date we complete our detailed review of your submittal, you must comply with both your existing Technical Specifications and your proposed inservice inspection and testing program. In the event conflicting requirements arise for some components, you must comply with the more restrictive requirements (e.g., shorter inspection intervals, increased number of parameters measured). In other words, the granting of this relief from ASME Code requirements, should not be interpreted to give you relief from any of the requirements in your existing Technical Specifications.

When our detailed review of your submittals is complete we will: (1) issue final approval of your program (which may contain modifications resulting from the staff's review) and (2) grant relief from any ASME Code requirements that are determined to be impractical for your facility for the duration of the inspection interval.

Sincerely,



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

cc: See next page

Mr. Dennis L. Farrar
Commonwealth Edison Company
Quad Cities Nuclear Power Station, Units 1 and 2

cc:

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