NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1): Peach Bottom Atomic Power Station - Unit 2

DOCKET NUMBER (2): 0 5 0 0 0 2 7 7

PAGE (3): 1 OF 0 3

TITLE (4): Reactor High Pressure Scram with all Rods Full-In

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	3 0	8 5	8 5	- 0 0 2	- 0 0	0 6	2 6	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	0 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Senior J. C. Nagle, Engineer - Special Projects	AREA CODE 2 1 5 / 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X											

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 2-85-02

During the vessel hydrostatic test, with all control rods in the full-in position and Peach Bottom Unit 2 in cold shutdown, the Reactor Protection System (RPS) initiated a scram signal on reactor high pressure during an excess flow check valve test on May 30, 1985. While performing surveillance test ST 13.8-1, "Excess Flow Check Valve Operability", a leak of approximately five liters per minute was discovered through the excess flow check valve on the recirculation pump seal cavity instrument sensing line. The leak caused reactor pressure to drop below the 1,000 pounds per square inch test pressure required by the surveillance test. The reactor operator responded to the decrease in reactor pressure by increasing control rod drive flow into the vessel to restore pressure. Simultaneously, test personnel isolated the leak by closing the instrument sensing line root valve. The reactor pressure increased rapidly to 1,030 pounds per square inch which resulted in the reactor high pressure scram signal.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Peach Bottom Atomic Power Station - Unit	0 5 0 0 0 2 7 7	8 5	0 0 2	0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of the Event:

During the vessel hydrostatic test, with all control rods in the full-in position and Peach Bottom Unit 2 in cold shutdown, the Reactor Protection System (RPS) initiated a scram signal on reactor high pressure during an excess flow check valve test on May 30, 1985. While performing surveillance test ST 13.8-1, "Excess Flow Check Valve Operability", a leak of approximately five liters per minute was discovered through the excess flow check valve on the recirculation pump seal cavity instrument sensing line. The leak caused reactor pressure to drop below the 1,000 pounds per square inch test pressure required by the surveillance test. The reactor operator responded to the decrease in reactor pressure by increasing control rod drive (CRD) flow into the vessel to restore pressure. Simultaneously, test personnel isolated the leak by closing the instrument sensing line root valve. The combined effects of increased CRD flow and isolation of the leak resulted in a rapid increase of reactor pressure to 1,030 pounds per square inch which resulted in the reactor high pressure scram signal. This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv). The EIIS code for the reactor protection system is JC.

Consequences of the Event:

At the time of this event, all control rods were fully inserted. The reactor protection system initiated the scram signal in accordance with Technical Specification requirements and the RPS logic functioned properly. Therefore, there were no adverse consequences.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Cause of the Event:

This event resulted from the test personnel responding to the leak in the excess flow check valve and the reactor operator attempting to maintain reactor pressure at the required surveillance test pressure of 1,000 pounds per square inch.

The reactor operator responded to the decrease in reactor pressure by increasing control rod drive flow into the vessel to restore pressure. As CRD flow was being increased the test personnel isolated the leak by closing the instrument sensing line root valve. With the reactor vessel hydraulically pressurized, the volume of CRD flow directed to the vessel caused a rapid pressure increase to the reactor high pressure scram setpoint before operator action could be taken.

Corrective Actions:

The scram signal was reset and surveillance testing of excess flow check valves successfully completed following repairs to the recirculation pump seal cavity instrument sensing line excess flow check valv[illegible].

Previous Similar Occurrences

None.

PHILADELPHIA ELECTRIC COMPANY

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PHILADELPHIA, PA. 19101

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June 26, 1985

Docket No. 50-277

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Peach Bottom Atomic Power Station - Unit 2

This LER concerns an automatic initiation of the reactor protection system with all control rods in the full-in position.

Reference:	Docket No. 50-277
Report Number:	2-85-02
Revision Number:	00
Event Date:	May 30, 1985
Report Date:	June 26, 1985
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,

W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator
Region I, USNRC

T. P. Johnson, Resident Inspector

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cc: Judge Helen F. Hoyt
Judge Jerry Harbour
Judge Richard F. Cole
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