



SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

RJR 85-292

June 17, 1985

DIRECTOR OF NUCLEAR REACTOR REGULATION
ATTN: HUGH L. THOMPSON, JR., DIRECTOR
DIVISION OF LICENSING
U S NUCLEAR REGULATORY COMMISSION
WASHINGTON DC 20555

DOCKET NO. 50-312
LICENSE NO. DPR-54
SUBJECT: PROPOSED AMENDMENT 122, REVISION 1


In accordance with 10 CFR 50.59, the Sacramento Municipal Utility District hereby proposes to amend its Operating License DPR-54 for Rancho Seco Nuclear Generating Station Unit No. 1. Per the requirements of 10 CFR 50.92, this Proposed Amendment 122, Revision 1, has been evaluated by a No Significant Hazards Consideration in Attachment 1.

This proposal incorporates the Once Through Steam Generator Inservice Inspection requirements from Proposed Amendment 63, Revision 2, dated February 19, 1985.

Attachment II to this submittal is a description of the proposed changes for Amendment 122, Revision 1.

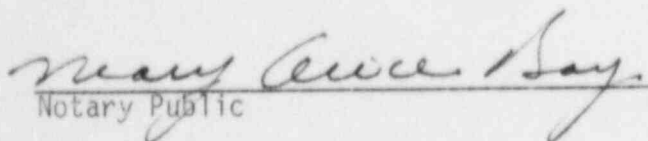
Since this is a revision to Proposed Amendment No. 122, no additional license fees are required.

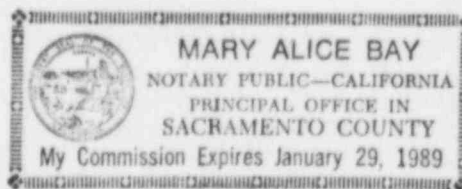
If you have any questions concerning this proposal, please contact Mr. Ron Colombo at Rancho Seco Nuclear Generating Station.


R. J. RODRIGUEZ
ASSISTANT GENERAL MANAGER, NUCLEAR

Enclosure

Subscribed and sworn to before me
this 21st day of June, 1985.


Notary Public



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ATTACHMENT I

NO SIGNIFICANT HAZARDS EVALUATION FOR PROPOSED AMENDMENT NUMBER 122, REVISION 1

BASIS FOR NO SIGNIFICANT HAZARDS CONSIDERATION

Description of Amendment Request

This Proposed Amendment 122, Revision 1, revises the Once Through Steam Generator Technical Specification by incorporating the OTSG requirements of Proposed Amendment 63, Revision 2. This proposal also follows up on the commitments made at the November 20, 1984 NRC meeting. The meeting minutes are included as supplemental information.

This amendment now contains two areas where the surveillance requirements have become more restrictive. First, the intent of NRC Regulatory Guide 1.83, Revision 1 has been met by including the Inservice Inspection requirements. The specific limited areas which are distinguished by unique operating conditions and/or physical construction will not be inspected for the purposes of meeting the random sample size requirements. Secondly, in the event of primary-to-secondary tube leakage in excess of the limits of Specification 3.1.6.9, an inspection of the affected steam generator will be performed in accordance with more rigid standards. If the leaking tube falls within one of the Special Area groups designated in either Table 4.17-3 or Table 4.17-4, all of the tubes in this group in this steam generator will be inspected. If the leaking tube does not fall within one of the Special Area Groups, an inspection will be performed on the affected steam generator in accordance with Table 4.17-2A with an initial inspection sample size of 6% of the tubes in the affected steam generator.

The Commission has provided guidance concerning the application of the standards for making a "No Significant Hazards Consideration" determination by providing certain examples (48FR14870). The district has reviewed these proposed changes with respect to the examples given and finds that the requested changes fall in the category of Commission examples.

- (i) "A purely administrative change to technical specifications: for example, a change to achieve consistency throughout the technical specifications, correction of an error, or a change in nomenclature."
- (ii) "A change that constitutes an additional limitation, restriction, or control not presently included in the technical specifications: for example, a more stringent surveillance requirement."
- (vii) "A change to make a license conform to changes in the regulations, where the license change results in very minor changes to facility operations clearly in keeping with the regulations."

Example (i) is correct since Proposed Amendment 122 editorially incorporates the Once Through Steam Generator Inservice Inspection requirements from Proposed Amendment 63, Revision 2. Example (ii) applies since the inspection sample size after a primary-to-secondary tube leak has been increased from 3% to 6% in Specification 4.17.3d2. Example (vii) also applies since the ISI requirements originated from NRC Regulatory Guide 1.83, Revision 1.

ATTACHMENT II

DESCRIPTION OF PROPOSED CHANGES

The following list describes changes between the original Proposed Amendment 122 and Revision 1.

1. List of Tables, page ix, Tables 4.17-2A, 2B; The original Table 4.17-2 has been expanded to include two tube inspection groups.
2. List of Tables, page ix, Tables 4.17-3, 4; These tables have been displaced back a page to account for the new Table 4.17-2B.
3. Page 4-51, Section 4.17.2; A statement has been added which describes the inspection result classification and the corresponding action required for the inspection of specific limited areas.
4. Page 4-52, Paragraph 4.17.2e; This is a new paragraph that deals with inspection requirements for tubes in specific limited areas.
5. Page 4-53, Paragraph 4.17.3b; Inservice Inspection frequency requirements are provided for Table 4.17-2A and Table 4.17-2B.
6. Page 4-53, Paragraph 4.17.3c; Table 4.17-2 has been changed to Table 4.17-2A.
7. Page 4-54, Paragraph 4.17.3d2; A sample size of 6% has been prescribed for inspections performed on the affected steam generator in accordance with Table 4.17-2A, given the leaking tube does not fall within one of the special area groups.
8. Page 4-54, Paragraph 4.17.4b; Table 4.17-2B and the provisions of Paragraph 4.17.2e have been incorporated into the definition of steam generator operability.
9. Page 4-55, Paragraph 4.17.5c; Specification 6.9 has been changed to Specification 6.9.5P.
10. Page 4-55, 1st Paragraph of the Bases; Included is a statement that the surveillance requirements are based on B&W - Standard Technical Specifications.
11. Page 4-55a, Last Paragraph of the Bases; Specification 6.9 has been changed to Specification 6.9.5P.
12. Page 4-57, Table 4.17.2A; This was formerly Table 4.17-2. In the Result Section C-3, Table 4.17-2 has been changed to Table 4.17-2A and Specification 6.9 has been revised to Specification 6.9.5P. Also, note (2) has been deleted.
13. Page 4-57a, Table 4.17-2B; This is a new table that provides the inspection result classification and the corresponding action required for inspection of special area tubes.

14. Table 4.17-3, Pages 4-57b, 57c, 57d, 57e; Previously this table occupied pages 4-57a, 57b, 57c, 57d.
15. Table 4.17-4, Page 4-57f; Page 4-57e was changed to Page 4-57f.
16. Page 6-12f, Specification 6.9.5P; Results of steam generator tube inspections which require notification of the Commission shall be reported pursuant to this Specification.